

Department of Environmental Quality
INL Oversight Program

**ENVIRONMENTAL SURVEILLANCE PROGRAM
QUARTERLY DATA REPORT**

January - March, 2022



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Table of Acronyms

aCi/L	-	attocuries per liter	nCi/L	-	nanocuries per liter
ATR	-	Advanced Test Reactor	NCRP	-	National Council on Radiation Protection and Measurements
BEA	-	Battelle Energy Alliance, LLC	NOAA	-	National Oceanic and Atmospheric Administration
BLR	-	Big Lost River	NRF	-	Naval Reactors Facility
CERCLA	-	Comprehensive Environmental Response, Compensation and Liability Act	PBF	-	Power Burst Facility
CFA	-	Central Facilities Area	pCi/g	-	picocuries per gram
CFR	-	Code of Federal Regulations	pCi/L	-	picocuries per liter
CITRC	-	Critical Infrastructure Test Range Complex	pCi/m ³	-	picocuries per cubic meter
DEQ-INL OP	-	The State of Idaho, Department of Environmental Quality, Idaho National Laboratory Oversight Program	QAPP	-	Quality Assurance Program Plan
DOE	-	U.S. Department of Energy	QA/QC	-	Quality Assurance/Quality Control
EBR I & II	-	Experimental Breeder Reactors I & II	RCRA	-	Resource Conservation and Recovery Act
EFS	-	Experimental Field Station	RPD	-	relative percent difference
EIC	-	electret ionization chamber	RTC	-	Reactor Technology Complex
EML	-	Environmental Monitoring Laboratory	RWMC	-	Radioactive Waste Management Complex
EPA	-	Environmental Protection Agency	SD	-	Sample standard deviation
ESER	-	Environmental Surveillance, Education and Research Program	SMC	-	Specific Manufacturing Capability
ESP	-	Environmental Surveillance Program	SMCL	-	secondary maximum contaminant level
ESRP	-	Eastern Snake River Plain	TAN	-	Test Area North
ESRPA	-	Eastern Snake River Plain Aquifer	TDS	-	total dissolved solids
Ft bls	-	feet below land surface	TMI	-	Three Mile Island
HPIC	-	high-pressure ion chamber	TRA	-	Test Reactor Area
IBL	-	Idaho Bureau of Laboratories	TSP	-	total suspended particulate
ICPP	-	Idaho Chemical Processing Plant	TSS	-	total suspended solids
ICP	-	Idaho Cleanup Project	USGS	-	U.S. Geological Survey
ISB	-	In-situ bioremediation	VOC	-	volatile organic compound
IDL	-	instrument detection limit	WLAP	-	Wastewater Land Application Permit
INL	-	Idaho National Laboratory			
INTEC	-	Idaho Nuclear Technology and Engineering Center			
ISU	-	Idaho State University			
LLD	-	lower limit of detection			
LSC	-	liquid scintillation counting			
MCL	-	maximum contaminant level			
MDA	-	minimum detectable activity			
MDC	-	minimum detectable concentration			
MFC	-	Materials and Fuels Complex			
µg/L	-	micrograms per liter			
mg/L	-	milligrams per liter			
MP	-	milepost			
mrem	-	millirem or 1/1000 th of a rem			
mR	-	milliRoentgen			
mR/hr	-	milliRoentgen per hour			
µR/hr	-	microRoentgen per hour			
MV	-	Magic Valley			
NIST	-	National Institute of Standards and Technology			

Introduction

The State of Idaho, Department of Environmental Quality, Idaho National Laboratory Oversight Program (DEQ-INL OP) conducts an Environmental Surveillance Program (ESP) at locations on the INL, near the boundaries of the INL, and at distant locations to the INL in accordance with accepted monitoring procedures and management practices. This program is designed to provide the people of the state of Idaho with independently evaluated information about the impacts of the Department of Energy's (DOE) activities in Idaho.

The primary objective for DEQ-INL OP's ESP is to maintain an independent environmental monitoring and verification program designed to verify and supplement DOE's environmental data and programs. This program also provides the citizens of Idaho with information on current and proposed DOE programs that has been independently evaluated to enable them to reach informed conclusions about DOE activities in Idaho and potential impacts to public health and the environment.

Results of the ESP are published using two distinct reporting formats: quarterly data reports and an annual ESP report. The annual ESP report is designed for a broad audience and summarizes the results of the ESP for the previous four quarters. The annual report's primary emphasis is to focus on trends, ascertain the impacts of DOE operations on the environment, and confirm the validity of DOE monitoring programs. This quarterly report is designed to document the results of the ESP on a quarterly basis and provide detailed data. It is organized according to the media sampled and also provides a quality assurance assessment.

Air and Precipitation Monitoring Results

The ESP operated eight air monitoring stations on and near the INL as well as two monitoring stations distant from the INL during the first quarter, 2022 (**Figure 1**). These stations employed instrumentation for collecting airborne particulate matter, gaseous radioiodine, precipitation, and water vapor for tritium analysis (**Table 1**). The Shoshone-Bannock Tribes operated an air monitoring station located at Fort Hall. The Fort Hall station uses identical instrumentation and sampling protocol as the ten stations operated by the ESP. The DEQ-INL OP reports the Fort Hall station data as an additional distant site.

Airborne particulate matter was sampled using both high-volume (8x10-inch filter) and low-volume (47-mm filter) total suspended particulate (TSP) air samplers. Many of these air samplers were found to be operating outside of their expected flow rate range in the first quarter, 2022. A calibrated flow rate gauge was taken into the field weekly to measure the sample start and stop flow rates of these samplers. The calibrated flow rate gauge measurements were used for the activity concentration calculations in the suspect measurements and have been footnoted as estimates. Weekly gross alpha and gross beta particulate radioactivity results for 47-mm filters from the low-volume TSP samplers are presented in **Appendix A** and summarized as a range of results in **Table 2**. Results are within the expected historical range.

Composites of 47-mm filters collected from low-volume TSP samplers during a calendar quarter are analyzed using gamma spectrometry. Composites of 8x10-inch filters collected from high-volume TSP samplers during each calendar month are also analyzed using gamma spectrometry. Typically, gamma spectrometry results are only reported when exceeding a minimum detectable concentration (MDC). Gamma spectrometry results for the first quarter of 2022 for 47-mm and 8x10-inch TSP filters are presented in **Tables 3** and **4**. For both filter sizes, the only reported gamma-emitting radionuclide was beryllium-7 (Be-7), a naturally occurring, cosmogenic radionuclide. The MDC for cesium-137 (Cs-137) is also reported since Cs-137 is the most likely of the man-made gamma emitting radionuclides to be detected.

Beginning in first quarter 2022, quarterly composites of 8x10-inch filters collected using high-volume TSP samplers are analyzed using radiochemical separation techniques. Results from the quarterly 8x10-inch filter composite analyses are typically presented in the following quarter's report. The samples are analyzed for Strontium-90, Plutonium-238, Plutonium-239/240, and Americium-241. Measurable quantities of these radionuclides are expected in the environment due to historic above ground testing of nuclear weapons, and possibly from INL programs. DEQ-INL OP's action levels of 19 for Americium-241 (Am-241), 190 for Strontium-90 (Sr-90), 21 for Plutonium-238 (Pu-238), and 20 for Plutonium-239/240 (Pu-239/240) (in 1×10^{-5} pCi/m³) are 10 percent of the compliance values listed for the specific radionuclides in 40 CFR 61, Appendix E, Table 2. Field sample concentrations which exceed these amounts require further investigation.

Radiochemical analyses results for composited 4-inch TSP filters from calendar year 2022 are presented in **Table 8**. The ISU vendor lab (ALS, Ft. Collins, CO.) non-conformance report listed significant quality control issues associated with the analyses of these samples, as discussed in the Quality Assurance section of this report. The original filter composite samples were completely consumed in the initial radiochemical separations, so repeat separations and analyses were not possible. The low tracer yields for Pu-238, Pu-239/240, and Am-241, and low chemical carrier recoveries for Sr-90 caused the MDCs to be elevated.

Three results exceed the Sr-90 MDC for the 2022 annual composite at Craters of the Moon, Experimental Field Station, and Van Buren Avenue. No results exceed the Pu-238 MDC. Two results exceed the Pu-239/240 MDC at Experimental Field Station and Fort Hall. One result is equal to the Am-241 MDC at Fort Hall. These greater-than-MDC results are all elevated compared to historical results and may suggest an INL source. However, the validity of these results is in doubt and their accuracy is questionable due to the vendor lab's quality control issues. Also, wood ash is known to contain the fallout radionuclides Sr-90 and Cs-137¹. It is possible that heavy smoke from wildfires in the third quarter of 2022 caused the elevated Sr-90 concentrations but an analysis of collected particulate matter mass data for the 2020 and 2022 filters was not informative. The questionable results from 2022 annual composites are all well below DEQ-INL OP action levels.

Preliminary radiochemical results from the first quarter 2022 8x10-inch filter quarterly composites are lower and in the range of historical values. Greater-than-MDC values range from 0.82 to 3.95×10^{-5} pCi/m³ for Sr-90, 0.03 to 0.04×10^{-5} pCi/m³ for Pu-238, and 0.02 to 0.03×10^{-5} pCi/m³ for Pu-239/240. There are no greater-than-MDC results for Am-241 (MDC range is 0.02 to 0.09×10^{-5} pCi/m³).

Radioactive iodine samples are collected weekly. Samples are collected by drawing air through a canister filled with activated charcoal using a low-volume air pump. The activated charcoal contained in the canister traps the radioiodine by adsorption onto its porous surface. Each week, canisters are collected from all eleven air monitoring stations and analyzed together as a composite using gamma spectrometry (**Table 5**). If Iodine-131 is detected in this grouping, the canisters are individually analyzed. No radioactive isotopes of iodine, specifically Iodine-131, were detected on the weekly charcoal cartridges used to collect this nuclide during the first quarter of 2022.

Atmospheric moisture was collected by drawing air through hygroscopic media at each of the 11 monitoring stations. This moisture was stripped from the hygroscopic media and analyzed to calculate the atmospheric tritium concentration. Reported values are the result of either a single sample or a weighted mean based upon the volume of air sampled when more than one atmospheric moisture sample was collected during the calendar quarter. All results are below MDCs and well below the DEQ-INL OP

¹ <https://www.burningissues.org/car-www/science/radwaste1.html>. Also (PDF) Cesium-137 in Wood Ash Results of Nationwide Survey (researchgate.net)

action level of 150 pCi/m³ (40 CFR 61). Atmospheric tritium concentrations and their weighted quarterly means are presented in **Table 6**.

Precipitation samples were collected at six monitoring locations during the first quarter of 2022. Precipitation samples were analyzed for tritium and man-made gamma emitting radionuclides. Reported values were either the result of a single sample or a weighted mean when more than one precipitation sample was collected during the calendar quarter. Tritium and man-made gamma emitting radionuclides were below minimum detectable concentration in precipitation collected during the first quarter of 2022. Analysis results for Tritium (H-3) and Cesium-137, the most likely to be detected of man-made gamma emitting radionuclides, are presented in **Table 7**.

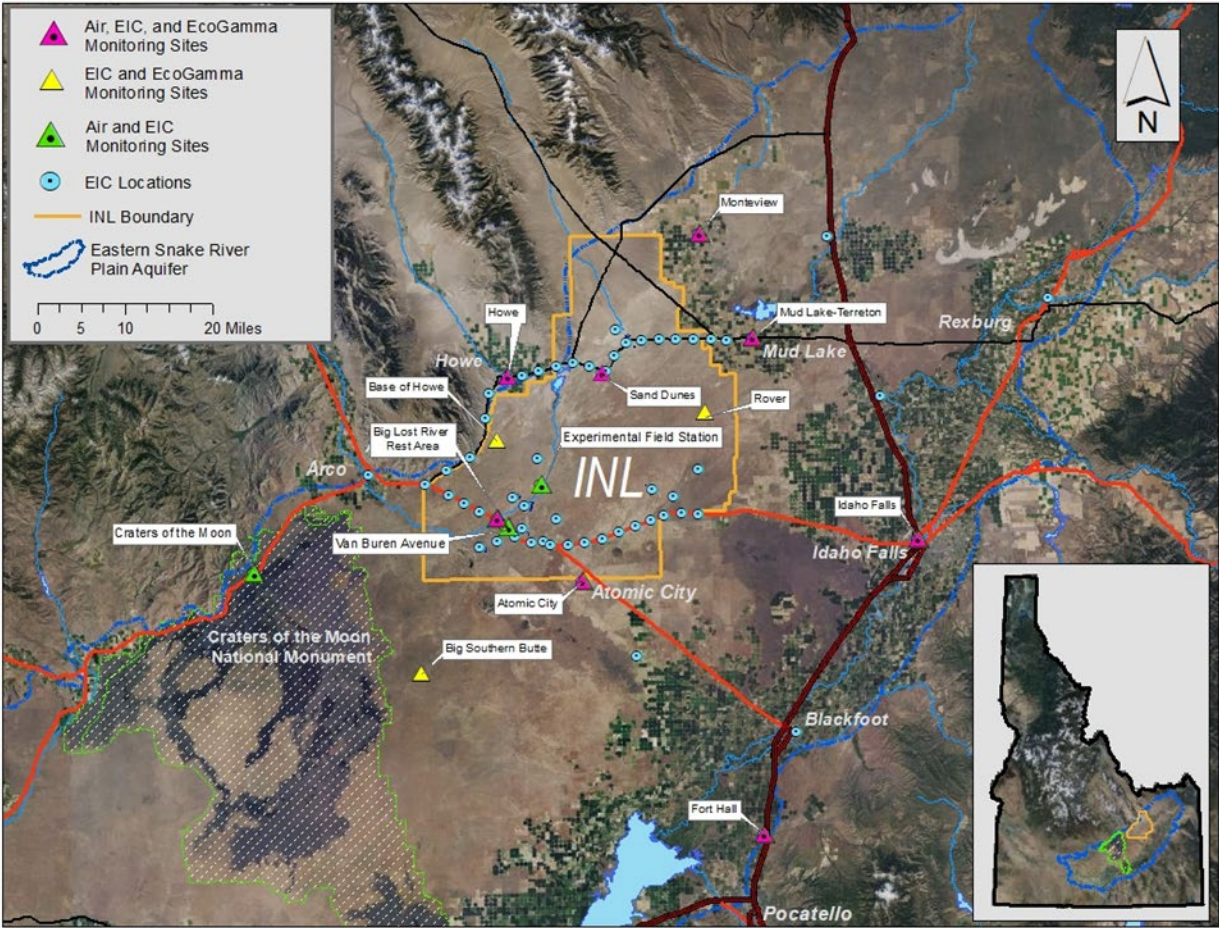


Figure 1. Air and radiation monitoring locations.

Table 1. Sampling locations and sample type.

Station Locations	Sample type ¹			
	TSP	Radioiodine	Water Vapor	Precipitation
On-site Locations				
Big Lost River Rest Area	☐	☐	■	■
Experimental Field Station	☐	☐	■	
Sand Dunes Tower	☐	☐	■	
Van Buren Avenue	☐	☐	■	
Boundary Locations				
Atomic City	☐	☐	■	■
Howe	☐	☐	■	■
Monteview	☐	☐	■	■
Mud Lake	☐	☐	■	■
Distant Locations				
Craters of the Moon	☐	☐	■	
Fort Hall ²	☐	☐	■	
Idaho Falls	☐	☐	■	■

¹☐ Samples collected weekly; ■ Samples collected quarterly.

²TSP and radioiodine samples collected by Shoshone-Bannock Tribes.

Table 2. Range of gross alpha and gross beta concentrations for TSP filters, first quarter, 2022.

Station Location	Concentration					
	Gross Alpha			Gross Beta		
On-Site Locations						
Big Lost River Rest Area	0.2	-	0.9	15.6	-	52.5
Experimental Field Station	0.1	-	1.0	16.9	-	59.4 J ²
Sand Dunes Tower	0.1	-	0.8	13.9	-	66.0
Van Buren Avenue	0.1	-	0.9 J ⁺³	15.9 J ²	-	48.6
Boundary Locations						
Atomic City	0.1	-	1.0	16.0	-	65.8
Howe	0.2	-	1.1 J ²	17.3	-	55.6 J ²
Monteview	0.2	-	1.2	15.7	-	63.1
Mud Lake	0.3	-	1.3	17.4	-	61.2
Distant Locations						
Craters of the Moon	-0.1	-	0.9 ³	13.2	-	39.0
Fort Hall ¹	0.1 J ²	-	1.1 J ^{+2,3}	10.2 J ²	-	34.3 J ²
Idaho Falls	0.2 J ²	-	1.0 J ^{+2,3}	15.7 J ²	-	48.6 J ²

¹Operated by Shoshone-Bannock Tribes.

²Air volume was estimated. Results are J-flagged as estimates.

³The TSP blank gross alpha result for the week of 2/16/22-2/23/22 minimally exceeded the MDC. The associated TSP gross alpha field result listed here was qualified as a biased-high estimate (J+).

Note: Concentrations are expressed in 1×10^{-3} pCi/m³.

Table 3. Gamma spectrometry analysis data for 47-mm TSP filters, quarterly composite samples, first quarter, 2022.

Station Location	Naturally Occurring Radionuclide Beryllium-7			Man-Made Gamma Emitting Radionuclides		
	Concentration	± 2 SD	MDC	Concentration	± 2 SD	MDC ²
On-site Locations						
Big Lost River Rest Area	54.6	3.9	2.4	0.07	0.08	0.13
Experimental Field Station	79.6 J ³	5.3 J	2.5 J	0.01 J	0.14 J	0.24 J
Sand Dunes Tower	102.0 J ³	7.0 J	3.4 J	-0.01 J	0.08 J	0.16 J
Van Buren Avenue	61.1 J ³	4.6 J	1.6 J	0.01 J	0.07 J	0.12 J
Boundary Locations						
Atomic City	79.8 J ³	5.0 J	1.8 J	0.04 J	0.07 J	0.12 J
Howe	63.1 J ³	4.2 J	1.5 J	0.00 J	0.06 J	0.10 J
Monteview	83.4	5.4	2.3	0.11	0.11	0.18
Mud Lake	61.6 J ³	4.2 J	1.4 J	0.03 J	0.06 J	0.10 J
Distant Locations						
Craters of the Moon	87.4	5.8	2.7	0.08	0.12	0.19
Fort Hall ¹	70.5 J ³	4.5 J	1.9 J	0.09 J	0.09 J	0.15 J
Idaho Falls	89.1 J ³	5.7 J	2.8 J	0.03 J	0.08 J	0.13 J

¹Operated by Shoshone-Bannock Tribes.

²MDC is for Cs-137. No man-made gamma emitting radionuclides were detected.

³Air volume was estimated in at least one weekly TSP sample for all locations. Results are J-flagged as estimates.

Note: Concentrations are reported in 1×10^{-3} pCi/m³ with associated uncertainty (± 2 SD) and minimum detectable concentration (MDC).

Table 4. Gamma spectrometry analysis data for 8x10-inch TSP filters, monthly composite samples, first quarter, 2022.

Station Location	Month	Naturally Occurring Radionuclide Beryllium-7			Man-Made Gamma Emitting Radionuclides		
		Concentration	± 2 SD	MDC	Concentration	± 2 SD	MDC ²
On-site Locations							
Big Lost River Rest Area	Jan	82.6	4.5	1.0	0.00	0.03	0.06
	Feb	93.0	5.3	0.7	0.03	0.04	0.06
	Mar ⁷	101.3	5.8	0.7	-0.01	0.03	0.05
Experimental Field Station	Jan	NS ⁴	NS	NS	NS	NS	NS
	Feb	82.0	4.8	0.6	0.02	0.03	0.06
	Mar ⁷	107.6	6.1	0.6	0.01	0.02	0.04
Sand Dunes Tower	Jan	90.1	5.3	0.8	0.04	0.04	0.06
	Feb	88.4	5.1	0.8	0.03	0.04	0.07
	Mar ⁷	108.1	6.1	0.6	0.02	0.02	0.04
Van Buren Avenue	Jan	88.5	4.8	1.1	-0.02	0.03	0.05
	Feb	79.7	4.6	0.7	0.02	0.03	0.05
	Mar ⁷	90.9	5.2	0.6	0.01	0.03	0.06
Boundary Locations							
Atomic City	Jan	104.8	6.0	0.8	-0.02	0.05	0.08
	Feb	92.6	5.4	0.7	0.01	0.04	0.06
	Mar ⁷	125.5	7.1	0.6	0.02	0.03	0.05
Howe	Jan ⁵	84.0	5.1	1.6	-0.03	0.08	0.14
	Feb	78.1	4.5	0.7	0.03	0.04	0.06
	Mar ⁷	99.2	5.6	0.7	0.02	0.03	0.04
Montevieu	Jan	97.4	5.3	1.0	-0.01	0.03	0.06
	Feb	98.9	5.8	0.8	-0.01	0.03	0.06
	Mar ⁷	107.9	6.2	0.6	0.00	0.02	0.04
Mud Lake	Jan ⁵	93.4	6.5	3.9	0.01	0.05	0.09
	Feb	77.7	4.5	0.8	0.01	0.04	0.06
	Mar ⁷	107.0	6.1	0.7	0.01	0.02	0.04
Distant Locations							
Craters of the Moon	Jan	91.3	5.3	0.7	0.01	0.03	0.04
	Feb	113.7	6.5	0.7	-0.01	0.05	0.09
	Mar ⁷	112.7	6.4	0.6	-0.01	0.02	0.04
Fort Hall ¹	Jan	NS ⁴	NS	NS	NS	NS	NS
	Feb ⁶	86.9	5.6	2.5	-0.04	0.12	0.20
	Mar ⁷	85.0	4.9	0.6	0.01	0.02	0.03
Idaho Falls	Jan	88.3	4.8	1.0	0.00	0.03	0.05
	Feb	86.9	5.1	0.8	-0.01	0.03	0.05
	Mar ⁷	116.8	6.6	0.6	0.00	0.03	0.05
Idaho Falls Duplicate ³	Jan	NS ⁴	NS	NS	NS	NS	NS
	Feb	82.9	4.8	0.8	0.02	0.03	0.05
	Mar ⁷	95.2	5.5	0.6	-0.01	0.02	0.04

¹Operated by Shoshone-Bannock Tribes.

²MDC is for Cs-137. No man-made gamma emitting radionuclides were detected.

³A duplicate 8x10-inch filter TSP sampler is currently being operated at the Idaho Falls location.

⁴NS – No sample. The Experimental Field Station and Idaho Falls Duplicate samplers were deployed on 1/26/22. The Fort Hall sampler was started on 2/15/22.

⁵Two filters/composite in January for Howe and Mud Lake. The Howe and Mud Lake samplers were deployed on 1/12/22. Other locations had four filters/composite.

⁶One filter in February for Fort Hall. Other locations had four filters/composite.

⁷Five filters/composite in March for all locations.

Note: Concentrations are reported in 1×10^{-3} pCi/m³ with associated uncertainty (± 2 SD) and minimum detectable concentration (MDC).

Table 5. Iodine-131 activity in weekly charcoal filter composites, first quarter, 2022.

Start Date	Collection Date	Iodine-131 activity (pCi/composite)		
		Activity	± 2 SD	MDA ¹
12/29/21	01/05/22	-0.31	1.34	2.32
01/05/22	01/12/22	-0.50	1.46	2.52
01/12/22	01/19/22	-0.19	1.28	2.22
01/19/22	01/26/22	-0.72	1.11	1.99
01/26/22	02/02/22	-0.93	1.46	2.55
02/02/22	02/09/22	-0.59	1.37	2.92
02/09/22	02/16/22	-0.24	1.11	1.93
02/16/22 ²	02/23/22	-1.03	2.07	3.55
02/23/22 ²	03/02/22	0.08	1.56	2.66
03/02/22	03/09/22	-0.10	1.35	2.31
03/09/22	03/16/22	-0.16	1.11	1.93
03/16/22	03/23/22	0.06	1.45	2.47
03/23/22	03/30/22	0.03	1.39	2.37

¹The minimum detectable activity (MDA) is established for the least efficient counting position in the eleven-cartridge composite.

²For the weeks beginning on 02/16/22 and 02/23/22, there were no cartridges from the Experimental Field Station sampler.

Based on a typical 20,000 ft³ (566 m³) air volume per cartridge, and only ten cartridges per composite for the week of 02/16/22, the highest I-131 MDA of 3.55 pCi/composite is equivalent to a maximum MDC of 6 x10⁻⁴ pCi/m³.

Table 6. Tritium concentrations in air from atmospheric moisture, first quarter, 2022.

Station Location	Start Date	Collection Date	Tritium		
			Concentration	± 2 SD	MDC
On-site Locations					
Big Lost River Rest Area	12/29/2021	03/09/2022	-0.02	0.21	0.35
Big Lost River Rest Area	03/09/2022	03/30/2022	0.23	0.38	0.63
Big Lost River Rest Area Mean	12/29/2021	03/30/2022	0.05	0.25	0.43
Experimental Field Station	12/29/2021	03/16/2022	0.30	0.22	0.35
Experimental Field Station	03/16/2022	03/30/2022	0.27	0.43	0.71
Experimental Field Station Mean	12/29/2021	03/30/2022	0.29	0.27	0.43
Sand Dunes Tower	12/29/2021	03/16/2022	0.14	0.22	0.37
Sand Dunes Tower	03/16/2022	03/30/2022	0.43	0.42	0.68
Sand Dunes Tower Mean	12/29/2021	03/30/2022	0.19	0.26	0.42
Van Buren Avenue	12/29/2021	02/23/2022	-0.06	0.21	0.37
Van Buren Avenue	02/23/2022	03/30/2022	0.01	0.29	0.49
Van Buren Avenue Mean	12/29/2021	03/30/2022	-0.03	0.24	0.42
Boundary Locations					
Atomic City	12/29/2021	02/23/2022	0.07	0.22	0.37
Atomic City	02/23/2022	03/30/2022	0.18	0.31	0.52
Atomic City Mean	12/29/2021	03/30/2022	0.11	0.25	0.42
Howe	12/29/2021	03/09/2022	-0.09	0.20	0.35
Howe	03/09/2022	03/30/2022	0.00	0.37	0.64
Howe Mean	12/29/2021	03/30/2022	-0.07	0.23	0.41
Mud Lake	12/29/2021	03/16/2022	0.21	0.22	0.36
Mud Lake	03/16/2022	03/30/2022	0.17	0.43	0.72
Mud Lake Mean	12/29/2021	03/30/2022	0.21	0.25	0.42
Montevieu	12/29/2021	03/30/2022	-0.05	0.23	0.39
Montevieu Mean	12/29/2021	03/30/2022	-0.05	0.23	0.39
Distant Locations					
Craters of the Moon	12/29/2021	02/16/2022	0.09	0.22	0.37
Craters of the Moon	02/16/2022	03/30/2022	0.10	0.27	0.46
Craters of the Moon Mean	12/29/2021	03/30/2022	0.10	0.25	0.42
Fort Hall ¹	12/29/2021	02/16/2022	-0.04	0.30	0.52
Fort Hall	02/16/2022	03/30/2022	0.04	0.32	0.55
Fort Hall Mean	12/29/2021	03/30/2022	0.00	0.31	0.53
Idaho Falls	12/29/2021	02/16/2022	-0.07	0.26	0.45
Idaho Falls	02/16/2022	03/30/2022	0.22	0.33	0.55
Idaho Falls Mean	12/29/2021	03/30/2022	0.06	0.29	0.49

¹Operated by Shoshone-Bannock Tribes.

Note: Concentrations are reported in pCi/m³ with associated uncertainty (± 2 SD) and minimum detectable concentration (MDC).

Table 7. Tritium and gamma-emitting radionuclide concentrations from precipitation, first quarter, 2022.

Station Location	Tritium			Cesium-137		
	Concentration	± 2 SD	MDC	Concentration	± 2 SD	MDC
On-site Locations						
Big Lost River Rest Area	40	100	160	-0.9	1.1	2.1
Boundary Locations						
Atomic City	-10	90	160	0.4	1.2	2.1
Howe	10	90	160	1.1	1.5	2.5
Monteview	70	100	160	0.8	1.5	2.6
Mud Lake	20	90	160	0.6	1.2	2.0
Distant Locations						
Idaho Falls	-40	90	160	-0.6	1.2	2.3

Note: Concentrations are reported in pCi/L with associated uncertainty (± 2 SD) and minimum detectable concentration (MDC).

Table 8. Annual radiochemical separation analysis data for 4-inch TSP particulate filters collected during 2021.

Station Location	⁹⁰ Sr			²³⁸ Pu			^{239/240} Pu			²⁴¹ Am		
	Value ¹	±2SD	MDC	Value ¹	± 2SD	MDC	Value ¹	±2SD	MDC	Value ¹	±2SD	MDC
On-Site Locations												
BLR ⁴ Rest Area J ⁶	2.01	3.55	5.80	0.03	0.06	0.10	0.03	0.06	0.10	0.03	0.03	0.13
EFS ³ J ⁶	9.0	4.58	6.42	0.03	0.16	0.09	0.18	0.16	0.09	-0.05	0.09	0.21
Sand Dunes J ⁶	5.36	4.76	7.50	-0.05	0.24	0.52	0.08	0.21	0.40	-0.05	0.15	0.36
Van Buren J ⁶	12.46	6.09	8.58	0.17	0.29	0.50	0.11	0.26	0.50	0.02	0.03	0.06
Boundary Locations												
Atomic City J ⁶	8.22	5.52	8.35	0.05	0.30	0.59	0.05	0.27	0.40	-0.01	0.06	0.13
Howe J ⁶	5.03	4.40	7.04	-0.52	0.77	1.81	0.00	0.63	1.38	0.10	0.60	1.25
Monteview J ⁶	5.73	4.63	7.28	0.08	0.19	0.34	0.08	0.21	0.40	0.06	0.33	0.65
Mud Lake J ⁶	0.24	2.74	4.64	0.07	0.09	0.13	0.07	0.09	0.13	0.03	0.04	0.06
Distant Locations												
Craters of Moon J ⁶	22.11	6.59	6.45	0.00	0.52	0.78	-0.11	0.52	0.99	0.10	0.52	0.82
Fort Hall ² J ⁶	6.36	6.36	10.1	0.03	0.07	0.13	0.07	0.05	0.03	0.05	0.04	0.05
Idaho Falls J ⁶	0.00	5.08	8.39	-0.12	0.20	0.44	0.17	0.23	0.38	0.01	0.03	0.07
Idaho Falls Dup J ⁶	7.27	5.96	9.30	-0.18	0.23	0.52	-0.08	0.21	0.48	-0.07	0.29	0.56

Note: Concentrations are reported in 1×10^{-5} pCi/m³ with associated uncertainty (± 2 SD), minimum detectable concentration (MDC), and correspond to filter composites collected during the calendar year.

¹ Measurable quantities of these radionuclides are expected in the environment due to historic above-ground testing of nuclear weapons, and possibly from INL programs. DEQ-INL OP's action levels of 19 for americium-241, 190 for strontium-90, 21 for plutonium-238, and 20 for plutonium-239/240 (in 1×10^{-5} pCi/m³) are 10 percent of the compliance values listed for the specific radionuclide in 40 CFR 61, Appendix E, Table 2.

² Operated by Shoshone-Bannock Tribes.

³ Experimental Field Station.

⁴BLR – Big Lost River.

⁵Dup – Duplicate TSP sampler being run at the Idaho Falls location in 2021.

⁶ The ISU vendor lab non-conformance report listed significant quality control issues associated with the analyses of these samples. Also, air volume was estimated in at least one weekly TSP sample for all locations. All values are therefore considered as estimates (J-flagged).

Environmental Radiation Monitoring Results

The ESP operated 13 environmental radiation stations during the first quarter of 2022 (**Figure 1**). To detect gamma radiation, each station is instrumented with triplicate electret ionization chambers (EIC), and 10 of the stations also are equipped with an EcoGamma gamma radiation monitor with low and high range Geiger-Müller detectors (**Table 8**).

The Shoshone-Bannock Tribes operate an air monitoring station at Fort Hall which is also equipped with EICs and an EcoGamma, both of which are owned and operated by the DEQ-INL OP. The DEQ-INL OP reports these results as a distant site.

EcoGammas are instruments capable of real-time measurements and are sensitive enough to detect small changes in gamma radiation levels. The real-time gamma radiation measurements collected by the EcoGammas at each location are transmitted to DEQ-INL OP and presented graphically via the worldwide web at <https://www.deq.idaho.gov/idaho-national-laboratory-oversight/inl-oversight-program/gamma-radiation-measurements>. Historically, DEQ-INL OP has used high-pressure ion chambers (HPIC) for real-time gamma radiation measurements. We have now completed a change-over of removing the old HPICs and replacing them with EcoGammas at each of our monitoring stations. Slight differences between EcoGamma data and historical HPIC data are expected.

EICs are a passive-integrating system that provides a cumulative measure of environmental gamma radiation exposure in the field. EICs are deployed, collected, and analyzed quarterly. EICs offer an inexpensive methodology to measure gamma radiation over a wide area, particularly in regions which do not have a power source. EICs can also provide valuable gamma radiation data in the event of an emergency. For this reason, EICs are deployed at 67 locations by DEQ-INL OP in a widespread network around the INL measuring external radiation. This information is tabulated in **Appendix B**.

These two systems are used by DEQ-INL OP to measure external gamma radiation for various radiological monitoring objectives. **Table 9** lists the average radiation exposure rates measured by the EcoGammas for first quarter 2022. **Table 10** lists the EIC monitoring results for first quarter 2022. Overall exposure rates were within the expected historical range of values observed by DEQ-INL OP for background radiation

Table 9. Summary of instrumentation at radiation monitoring stations.

Station Location	Instrument Type	
	EcoGamma	EIC
On-site Locations		
Base of Howe	■	■
Big Lost River Rest Area	■	■
Experimental Field Station		■
Rover	■	■
Sand Dunes Tower	■	■
Van Buren Avenue		■
Boundary Locations		
Atomic City	■	■
Big Southern Butte	■	■
Howe Met Tower	■	■
Monteview	■	■
Mud Lake/Terreton	■	■
Distant Locations		
Craters of the Moon		■
Fort Hall	■	■
Idaho Falls	■	■

Table 10. Average gamma exposure rates, first quarter, 2022, EcoGamma* network.

Station Location	Exposure Rate (µR/hr)	
	Quarterly Average	± 2 SD
On-site Locations		
Base of Howe	16.7	1.1
Big Lost River Rest Area	14.7	0.8
Rover ¹	-	-
Sand Dunes Tower	15.1	0.8
Boundary Locations		
Atomic City	14.2	0.9
Big Southern Butte	16.2	1.4
Big Southern Butte Duplicate ²	12.7	1.0
Howe Met Tower	14.1	0.8
Monteview	12.1	0.8
Mud Lake / Terreton	14.1	0.8
Distant Locations		
Fort Hall	13.8	0.7
Idaho Falls	15.5	0.8

*The EcoGammas are sensitive electronic devices that can experience intermittent malfunctions and/or interference; this typically results in characteristic positive and/or negative data spikes. These aberrations are removed from the data set based on the judgement of the data analyst.

¹No data for this quarter.

²A duplicate EcoGamma is installed at Big Southern Butte for testing.

Table 11. Electret ionization chamber (EIC) cumulative average exposure rates, first quarter, 2022.

Station Location	Exposure Rate (µR/hr)	
	Quarterly Average ¹	± 2 SD
On-Site Locations		
Base of Howe	11.5	0.5
Big Lost River Rest Area	12.8	1.7
Experimental Field Station	11.7	2.7
Rover	14.2	1.6
Sand Dunes Tower	12.0	2.1
Van Buren Avenue	20.4, 20.8	-
Boundary Locations		
Atomic City	12.7	1.4
Big Southern Butte	13.3	2.2
Howe Met Tower	13.5	4.6
Monteview	14.1	3.2
Mud Lake/Terreton	13.6	5.4
Distant Locations		
Craters of the Moon	11.2	4.9
Fort Hall	8.5, 9.7	-
Idaho Falls	10.3	1.5

¹Results are the average of triplicate exposure rate measurements with the associated sample variability (±2 SD), or the 2 measured exposure rates remaining after removal of an outlying value. One of the triplicate measurements is rejected if it is outside the average of the triplicate measurements ±2 SD of the historical population variability. Typically, the two most consistent measurements are reported, based on judgment of the data analyst.

Water Monitoring Results

DEQ-INL OP collects groundwater samples from wells and springs located within, upgradient of, and downgradient of the INL to evaluate the effects of INL contaminants on water quality in the eastern Snake River Plain (ESRP) aquifer and verify the results of DOE and USGS monitoring. Each year, DEQ-INL OP samples approximately 85-90 locations concurrently with a DOE contractor or the USGS and 15-20 locations independently. Co-sampled locations are primarily on or near the INL Site and are usually sampled during the second and fourth calendar quarters. DEQ-INL OP publishes a comparison of its own analytical results with those obtained by co-samplers in the DEQ-INL Oversight Program Annual Report. Locations sampled independently by DEQ-INL OP are mostly in the Magic Valley and are typically sampled during the third calendar quarter.

Most water samples are collected from wells drilled into the aquifer or springs formed by the intersection of the aquifer water table with the surface. Each aquifer well or spring is categorized as upgradient, facility, boundary, or distant based on its location (**Figure 2** and **Figure 3**):

- *Upgradient* sites are situated north or northeast of INL facilities in areas that have not been affected by INL operations. They are used to monitor background concentrations in the aquifer.
- *Facility* sites are located near facility complexes within the INL, including the Advanced Test Reactor complex (ATR), the Central Facilities Area (CFA), the Idaho Nuclear Technology and Engineering Center (INTEC), the Materials and Fuels Complex (MFC), the Naval Reactors Facility (NRF), the Radioactive Waste Management Complex (RWMC), and Test Area North (TAN). Facility sites are located within or immediately downgradient of known areas of

contamination and are sampled to monitor the concentrations and migration of specific contaminants.

- *Boundary* sites are located near the southern boundary of the INL, downgradient of potential sources of INL contamination. These include several wells equipped with Westbay Multilevel Groundwater Monitoring Systems (“Westbay wells”), which offer a look at the vertical distribution of constituents in the aquifer.
- *Distant* sites are located farther downgradient of the INL, primarily in the Magic Valley, and include wells and springs used for agricultural, municipal, domestic, and industrial purposes.

A small number of samples are also collected each year from streams, waste-pond effluent, and wells drilled into perched groundwater (groundwater that sits above the aquifer).

Samples collected from water-monitoring sites are analyzed for radiological and non-radiological constituents, many of which are present in the aquifer both naturally and as a result of INL operations. All locations are sampled for gross alpha and gross beta radioactivity, manmade gamma-emitting nuclides, tritium, chloride, chromium, and nitrate-plus-nitrite.² Samples from locations at which tritium concentrations are too low to be detected by the standard method are re-analyzed for tritium using an electrolytic enrichment method (referred to as the low-level method), which has a minimum detectable concentration (MDC) about ten times lower than the standard method. Selected sites are also sampled for specific radionuclides—including uranium isotopes (²³⁴U, ²³⁵U, and ²³⁸U), plutonium isotopes (²³⁸Pu, ^{239/240}Pu), americium-241 (²⁴¹Am), strontium-90 (⁹⁰Sr), and technetium-99 (⁹⁹Tc)—selected trace metals, common ions, total phosphorous, and/or volatile organic compounds (VOCs) based on past and present INL operations or a history of elevated concentrations. If unexpected levels of radioactivity are detected in gross measurements, additional samples will be collected and analyzed for specific radionuclides.

During the first quarter of 2022, DEQ-INL OP sampled groundwater from the aquifer at 6 facility locations. **Table 12** lists the sample date, co-sampler, well depth, and analyses requested for the locations sampled this quarter. Analytical results are reported in **Tables 14 through 22** and summarized below.

Table 13 shows the range of background concentrations for each constituent in the ESRP aquifer and the EPA drinking water maximum contaminant level (MCL) or secondary maximum contaminant level (SMCL). Background concentrations depend on local geology, and the concentrations of constituents at sites not influenced by INL activities may on occasion be higher than the given background ranges due to local factors and natural variability.

Gross alpha and gross beta radioactivity

Gross alpha and gross beta analyses are used to screen for unexpectedly high levels of radioactivity in samples. DEQ-INL OP has determined from past sampling that background concentration ranges for gross alpha and gross beta radioactivity in the ESRP aquifer are approximately 0-5.6 pCi/L and 0-8.6 pCi/L, respectively. Occasional measurements of concentrations above these background ranges in uncontaminated samples are statistically probable due to uncertainties inherent in measuring low levels of radioactivity. Additionally, some samples will have levels of radioactivity slightly higher than background ranges due to higher-than-average concentrations of naturally occurring uranium, thorium, or potassium-40.

² Distant locations Alpheus Spring, Bill Jones Hatchery, Clear Spring, Minidoka Water Supply, and Shoshone Water Supply and upgradient location Mud Lake Water Supply are sampled for gross alpha and gross beta radioactivity, gamma-emitting radionuclides, and tritium during the second quarter. In the fourth quarter, samples are collected for common ions, metals, nitrate-plus-nitrite, and other constituents along with gross alpha and gross beta radioactivity, gamma-emitting radionuclides, and tritium.

Six facility locations were analyzed for gross alpha and beta radioactivity this quarter. (**Table 14**). Gross alpha radioactivity was measured at concentrations within the known background range at all locations. In contrast, elevated gross beta concentrations were measured at all but one location. TAN-2336 had the highest gross beta concentration value at 1601.5 ± 48.9 pCi/L, which is consistent with the well's elevated ^{90}Sr results. At INTEC, an elevated gross beta concentration result of 720.6 ± 6.3 pCi/L was measured at ICPP-MON-A-230. Concentrations are consistent with previous results. The gross beta values at USGS-052 this quarter at 246.2 ± 3.5 pCi/L were elevated from the 2021 values of 199.9 ± 3.3 pCi/L. The elevated gross beta concentrations are likely due to increasing ^{99}Tc values. In 2021, ^{99}Tc concentration values for USGS-052 were 316 ± 53 pCi/L. This quarter they increased to 337 ± 56 pCi/L (original sample) and 401 ± 67 pCi/L (duplicate). We will continue to watch this well for an upward trend. All other detectable concentrations in groundwater were consistent with historical data and were measured in areas of known contamination related to past INL waste disposal practices.

Manmade gamma-emitting radionuclides

There was just one location sampled this quarter where manmade gamma-emitting radionuclides were detected: TAN-2336 had a cesium-137 concentration of 6.1 ± 1.9 pCi/L. Although the values increased from 3.0 ± 1.8 pCi/L in 2021, they are well below the drinking water MCL of 200 pCi/L for Cs-137. All results for cesium-137 (^{137}Cs), the manmade gamma-emitter most likely to be detected in groundwater, are reported in **Table 14**.

Tritium

Tritium was analyzed for all INTEC and TAN aquifer locations sampled this quarter (**Table 15**) with elevated tritium concentrations observed in all the samples. The highest tritium concentration was observed at INTEC well USGS-123 at 1552 ± 160 pCi/L. Elevated tritium concentrations (475 ± 110 pCi/L) were also present at TAN-2336.

Results from the wells are consistent with historical trends and all tritium concentrations reported in this quarter are below the drinking water MCL of 20,000 pCi/L. No low-level tritium samples were taken this quarter and there is currently no backlog for low-level tritium analyses.

Strontium-90

All six aquifer facility wells (INTEC and TAN) were sampled for ^{90}Sr this quarter (**Table 16**) and all samples with exception of USGS-123, had detectible concentrations. Concentrations ranged from 1.93 ± 0.51 pCi/L at USGS-052 to 590 ± 140 pCi/L at TAN-2336. TAN-2336, USGS-047, and USGS-067 exceeded the drinking water MCL of 8 pCi/L; however, all results were within historical ranges.

In the Fall of 2021, boundary well USGS-104 was sampled for ^{90}Sr with results of 1.03 ± 0.42 pCi/L (MDC = 0.63). The historical range of values for this location has been approximately 0 – 0.2 pCi/L (less-than-MDC values). The lab did not indicate problems with the analysis yielding the elevated values and the gross beta results for USGS-104 were consistent with past results at 2.4 ± 0.9 pCi/L; no reason was found to reject the elevated results. Due to the increased ^{90}Sr values, a re-analysis of the original sample along with a re-analysis of the original sample for CFA-2, located directly upstream from USGS-104, were requested from the lab. The re-analyses resulted in non-detects for both samples with USGS-104 at 0.05 ± 0.12 pCi/L (MDC = 0.23 pCi/L) and CFA-2 at 0.01 ± 0.13 pCi/L (MDC = 0.25 pCi/L). Routine sampling of USGS-104 and CFA-2 is scheduled to occur in the Fall of 2022. Future results will be carefully reviewed.

Technetium-99

All five of the INTEC locations were sampled for ^{99}Tc (**Table 17**) and all results were consistent with historical data. ICPP-MON-A-230, USGS-052, and USGS-067 had detectable concentrations, with the highest concentrations of 1310 ± 210 pCi/L at ICPP-MON-A-230. This is a slight decrease from concentrations of 1510 ± 240 pCi/L observed in 2021. With exception of ICPP-MON-A-230, all concentrations were below the drinking water MCL of 900 pCi/L.

Iodine-129

Three INTEC locations (USGS-047, USGS-067, and USGS-123) were sampled for ^{129}I ; however, the results from the lab are not yet available. Once available, results will be published in a future quarterly report.

Actinides

One location at TAN (TAN-2336) was sampled for uranium isotopes this quarter (**Table 18**). Results for ^{234}U and ^{238}U measured within background concentration levels at 0.94 ± 0.26 pCi/L and 0.17 ± 0.09 pCi/L, respectively. An elevated $^{234}\text{U}/^{238}\text{U}$ activity ratio of 5.5 suggests that the source was anthropogenic. Results for ^{235}U were elevated above background levels at 0.09 ± 0.074 pCi/L; however, this result is considered an estimate (J-flagged) because it is less than three sample standard deviations. All results for uranium isotopes were consistent with previous results.

Common ions, trace metals, and nutrients

All five INTEC locations were sampled for chloride, chromium, sulfate, alkalinity, and dissolved nutrients (nitrate-plus-nitrite). One TAN location was sampled for common ions, trace metals, nutrients (nitrate-plus-nitrite), and phosphorus. (**Tables 19, 20, and 21**).

TAN-2336 exceeded most background levels for common ions, trace metals, and phosphorus, which is likely due to the on-going bioremediation well injections. TAN-2336 had the highest concentrations above background for chloride at 125 mg/L and alkalinity at 5660 mg/L. The alkalinity results were much higher than reported for TAN-2336 in 2021 at 1260 mg/L. INTEC well ICPP-MON-A-230 had slightly elevated concentrations of chloride (74.0 mg/L) above background. All other locations' results for common ions were within background concentration levels.

TAN-2336 exceeded background levels for trace metals: barium (1600 $\mu\text{g/L}$, MCL is 2000 $\mu\text{g/L}$), chromium (130 $\mu\text{g/L}$, MCL is 100 $\mu\text{g/L}$), iron (16000 $\mu\text{g/L}$, SMCL is 300 $\mu\text{g/L}$), and manganese (2200 $\mu\text{g/L}$, SMCL is 50 $\mu\text{g/L}$). ICPP-MON-A-230, USGS-123, USGS-047, USGS-052, and USGS-067 had slightly elevated concentrations of chromium above background; however, these values were similar to past values.

ICPP-MON-A-230 exceeded background concentration levels for nutrients ($\text{NO}_3 + \text{NO}_2$) at 6.8 mg/L, however, this value is slightly lower than the 2021 value of 7.3 mg/L. USGS-067 exceeded background concentration levels for nutrients at 5.2 mg/L. This value is essentially the same as the 2021 value of 5.3 mg/L. The other wells sampled for nutrients were within the background range (<0.04 – 3.59). The only well that was sampled for phosphorus this quarter was TAN-2336, which resulted in a phosphorus concentration above background at 11 mg/L.

Volatile organic compounds (VOCs)

VOCs were measured at TAN-2336 (**Table 22**). Notable MCL exceedances and/or changes from previous measurements include:

- TAN-2336: MEK (2-Butanone) = 1140 µg/L, up from <0.5 µg/L in 2021
- TAN-2336: Methylene chloride = 21.2 µg/L, up from <0.5 µg/L in 2021

During the fermentation process that results from the bioremediation well injections conducted at TAN-2336, significant levels of MEK (2-Butanone) are produced. The levels are transient and are expected to rapidly dissipate once the bioremediation injections are complete. An article written by Fowler, Thompson, Mueller, 2011 discussing this process, can be found at: <https://doi.org/10.1002/rem.21296>. The VOCs detected above do not have established EPA MCLs. No other VOCs were detected this quarter. Appendix C provides a list of VOCs analyzed for water samples.

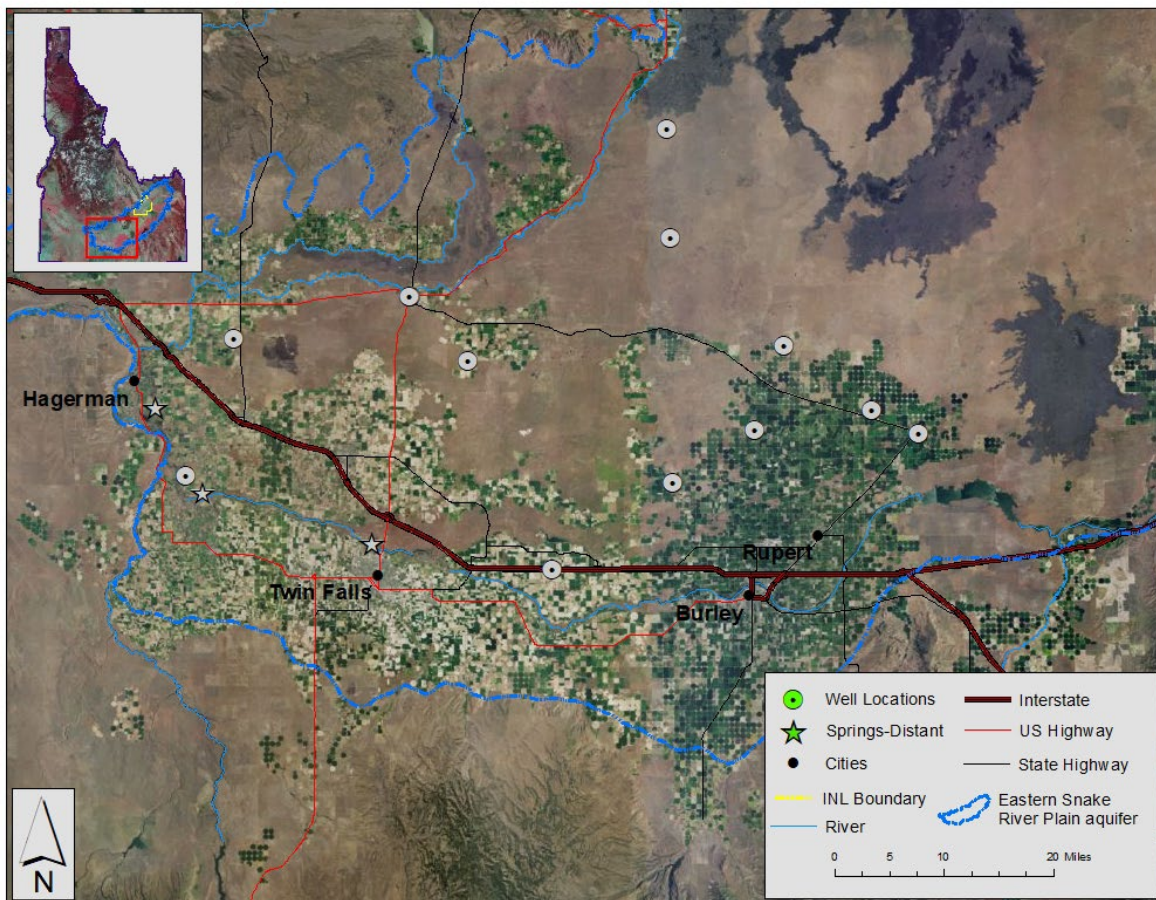


Figure 2. Distant water monitoring locations. No distant water monitoring sites were sampled during reporting period.

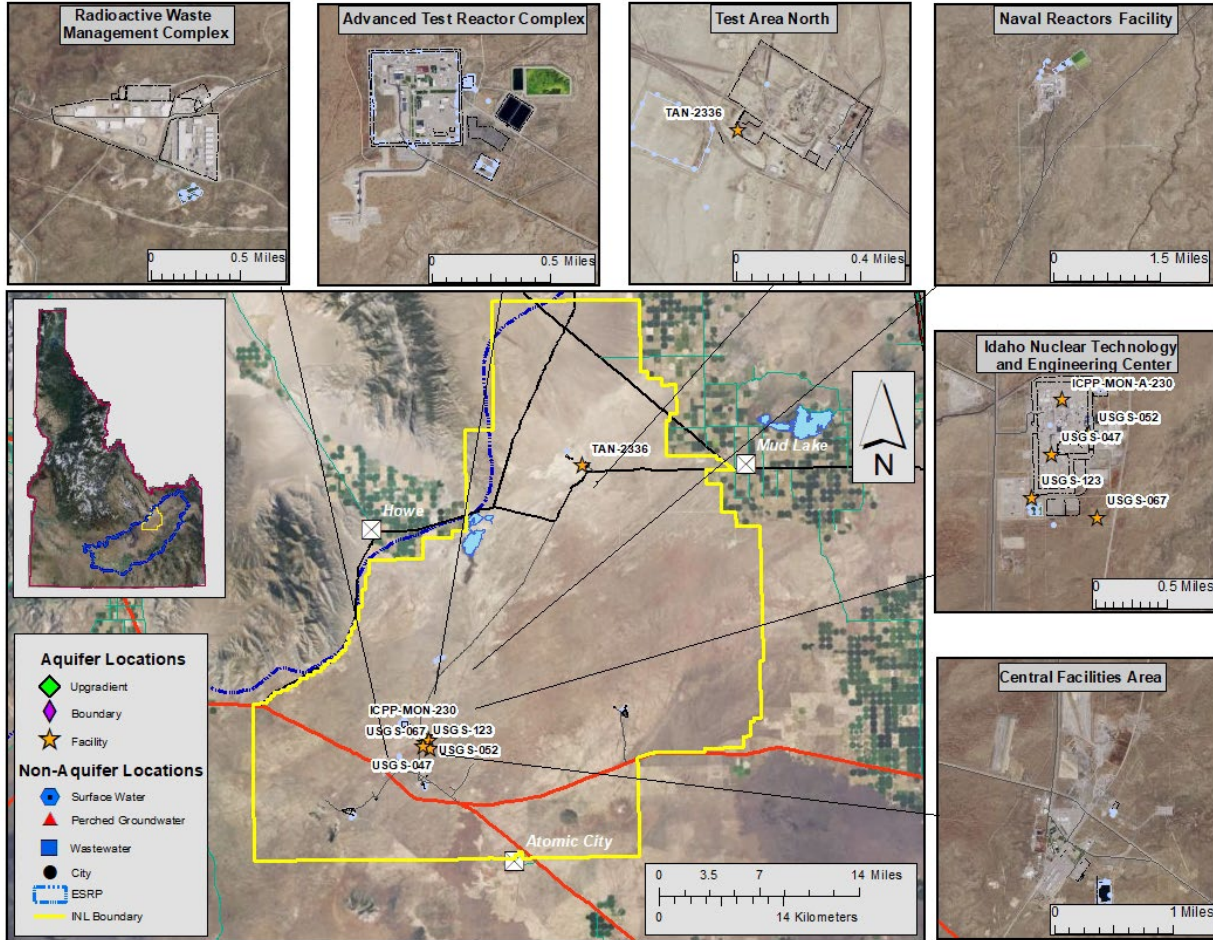


Figure 3. Up-gradient, facility, boundary, perched groundwater (GW), surface water, and wastewater monitoring locations.

Table 12. Locations sampled for water, first quarter, 2022.

Sample Location	Date Sampled	Co-sampler	Well Depth (ft bgs)	Analyses*
Aquifer Samples				
Facility				
<i>Idaho Nuclear Technology and Engineering Center:</i>				
ICPP-MON-A-230	03/28/2022	IEC	503	α , β , γ , ^3H , ^{90}Sr , ^{99}Tc , Cl^- , SO_4^{2-} , Cr , NO_3+NO_2
USGS-047	03/28/2022	IEC	488	α , β , γ , ^3H , ^{90}Sr , ^{99}Tc , ^{129}I , Cl^- , SO_4^{2-} , Cr , NO_3+NO_2
USGS-052	03/29/2022	IEC	461	α , β , γ , ^3H , ^{90}Sr , ^{99}Tc , Cl^- , SO_4^{2-} , Cr , NO_3+NO_2
USGS-067	03/29/2022	IEC	685	α , β , γ , ^3H , ^{90}Sr , ^{99}Tc , ^{129}I , Cl^- , SO_4^{2-} , Cr , NO_3+NO_2
USGS-123	03/02/2022	IEC	474	α , β , γ , ^3H , ^{90}Sr , ^{99}Tc , ^{129}I , U iso. , Cl^- , SO_4^{2-} , Cr , NO_3+NO_2
<i>Test Area North:</i>				
TAN-2336	01/11/2022	IEC	255	α , β , γ , ^3H , ^{90}Sr , U iso. , com. ions, trace metals, NO_3+NO_2 , P , VOCs

ft bgs = feet below ground surface.

* α = gross alpha radioactivity; β = gross beta radioactivity; γ = manmade gamma-emitting radionuclides; ^3H = tritium; ^{90}Sr = Strontium-90; ^{99}Tc = Technetium-99; ^{241}Am = Americium-241; P iso. = ^{238}Pu , $^{239/240}\text{Pu}$; U iso. = ^{234}U , ^{235}U , ^{238}U ; Cl^- = chloride; Cr = chromium; com. ions = calcium (Ca^{2+}), magnesium (Mg^{2+}), sodium (Na^+), potassium (K^+), chloride (Cl^-), sulfate (SO_4^{2-}), alkalinity; trace metals = arsenic (As), barium (Ba), chromium (Cr), iron (Fe), manganese (Mn), lead (Pb); NO_3+NO_2 = nitrate plus nitrite; P = phosphorus, and VOCs = volatile organic compounds.

IEC = Idaho Environmental Coalition, LLC - This is a contractor that replaced Fluor Idaho, LLC in 2021.

Table 13. Constituent background concentration ranges and EPA drinking water standards.

Constituent	Background ¹	MCL or SMCL ²
Radiological Constituents (pCi/L)		
Gross alpha	0-5.6 ^a	15
Gross beta	0-8.6 ^a	4 mrem/yr
Cesium-137	0	200
Tritium	0-33 ^a	20,000
Strontium-90	0	8
Technetium-99	0	900
Uranium-234	0.043-1.9 ^{b,3}	30 µg/L (total U)
Uranium-235	0-0.048 ^b	
Uranium-238	0.021-0.719 ^{b,3}	
Plutonium-238	0	---
Plutonium-239/240	0	---
Americium-241	0	---
Non-radiological Constituents		
<i>Common Ions (mg/L)</i>		
Alkalinity (as CaCO ₃)	91-261 ^a	---
Calcium	23 – 71 ^a	---
Chloride	4.9 – 66.6 ^a	250*
Fluoride	0.1 – 1.50 ^a	4
Magnesium	10.1 – 27.4 ^a	---
Potassium	1.2 – 5.8 ^a	---
Sodium	2.6 – 27.0 ^a	---
Sulfate	9.6 – 40.4 ^a	250*
<i>Trace Metals (µg/L)</i>		
Arsenic	2 – 3 ^c	10
Barium	50 – 70 ^c	2000
Chromium	<1.0 – 5.2 ^a	100
Iron	4 – 16 ^d	300*
Lead	<5 ^c	15
Manganese	<1 – 4 ^a	50*
Selenium	<1 ^c	50
Zinc	<3 – 10.5 ^d	5000*
<i>Nutrients (mg/L)</i>		
Nitrate plus nitrite	<0.04 – 3.59 ^b	10 for NO ₃ ⁻ , 1 for NO ₂ ⁻
Phosphorus	<0.01 – 0.02 ^d	---
<i>Volatile Organic Compounds (µg/L)</i>		
Tetrachloroethene (PCE)	0	5
Trichloroethene (TCE)	0	5
1,1-Dichloroethene	0	7
cis-1,2-dichloroethene	0	70
trans-1,2-dichloroethene	0	100
Vinyl chloride	0	2
Carbon tetrachloride	0	5
Chloroform	0	80 ^e
Chloromethane	0	---
Methylene Chloride	0	5
Methyl Ethyl Ketone	0	---
1,1-Dichloroethane	0	---

¹ Sources for background ranges are: ^a DEQ data compiled from distant, boundary, and surface water sites from 1993-2018.

^b Bartholomay and Hall, 2016 (DOE/ID-22237); ^c Knobel and others, 1992; ^d Knobel and others, 1999 (DOE/ID-22164).

² Maximum Contaminant Levels (MCLs) are the highest levels of contaminants legally allowed in public drinking water systems in Idaho. Most wells sampled by DEQ-INL OP are not used for drinking water. A * designates a Secondary MCL (SMCL), which is a guideline recommended by the EPA for constituents that may affect the taste, color, or odor of drinking water. ^e MCL is for total trihalomethanes.

³ESRPA samples containing only naturally occurring uranium should have a ²³⁴U/²³⁸U activity ratio of about 1.5 to 3.1.

Table 14. Gross alpha, gross beta, and man-made gamma-emitting radionuclide concentrations (pCi/L) in water samples, first quarter, 2022.

Sample Location	Sample Date	Gross Alpha			Gross Beta			Cesium-137*		
		Concentration	2 SD		Concentration	2 SD		Concentration	2 SD	
Aquifer Samples										
Facility										
<i>Idaho Nuclear Technology and Engineering Center</i>										
ICPP-MON-A-230	03/28/2022	5.2	-	1.5	720.6	-	6.3	0.4	U	1.3
USGS-047	03/28/2022	1.7	-	0.8	33.7	-	1.5	2.1	U	1.4
USGS-052	03/29/2022	-0.5	U	0.8	246.2	-	3.5	0.3	U	1.3
USGS-067	03/29/2022	2.2	-	0.9	106.6	-	2.5	0.8	U	1.1
USGS-123	03/02/2022	2.0	-	0.7	2.5	-	0.9	-0.4	U	1.1
Test Area North										
TAN-2336	01/11/2022	2.7	U	24.8	1601.5	-	48.9	6.1	-	1.9

Data qualifiers: U = undetected, J = estimate, R = rejected, "+" or "-" after a J means that the estimated result is biased high or low, respectively.
 *ISU-EML analyzes water samples for all common manmade gamma-emitting radionuclides. If none are detected, only the results for ¹³⁷Cs, the manmade gamma-emitter most likely to be detected in groundwater, are reported in this table.
 Typical MDC range (gross alpha) 0.7 – 1.9 pCi/L. Typical MDC range (gross beta) 1.2 – 1.4 pCi/L. Higher 2 SDs and MDCs for TAN-2336 were a result of high suspended/dissolved solids requiring a smaller sample for evaporation prior to counting. Typical MDC range (Cs-137) 1.8 – 2.7 pCi/L.

Table 15. Tritium concentrations (pCi/L) in water samples, first quarter, 2022.

Sample Location	Sample Date	Tritium		
		Concentration	2 SD	
Aquifer Samples				
Facility				
<i>Idaho Nuclear Technology and Engineering Center</i>				
ICPP-MON-A-230	03/28/2022	695	-	120
USGS-047	03/28/2022	342	-	110
USGS-052	03/29/2022	313	-	120
USGS-067	03/29/2022	1543	-	150
USGS-123	03/02/2022	1552	-	160
Test Area North				
TAN-2336	01/11/2022	475	-	110

Data qualifiers: U = undetected, J = estimate, R = rejected, "+" or "-" after a J means that the estimated result is biased high or low, respectively. MDC range 150 - 170 pCi/L.

Table 16. Strontium-90 concentrations (pCi/L) in water samples, first quarter, 2022.

Sample Location	Sample Date	Strontium-90		
		Concentration	2 SD	
Aquifer Samples				
Facility				
<i>Idaho Nuclear Technology and Engineering Center</i>				
ICPP-MON-A-230	03/28/2022	2.31	-	0.62
USGS-047	03/28/2022	11.8	-	2.8
USGS-052	03/29/2022	1.93	-	0.54
USGS-067	03/29/2022	9.2	-	2.2
USGS-123	03/02/2022	0.50	U	0.38
Test Area North				
TAN-2336	01/11/2022	590	-	140

Data qualifiers: U = undetected, J = estimate, R = rejected, "+" or "-" after a J means that the estimated result is biased high or low, respectively.
 MDC range 0 – 0.75 pCi/L.

Table 17. Technetium-99 concentrations (pCi/L) for water samples, first quarter, 2022.

Sample Location	Sample Date	Technetium-99		
		Concentration		2 SD
Aquifer Samples				
Facility				
<i>Idaho Nuclear Technology and Engineering Center</i>				
ICPP-MON-A-230	03/28/2022	1310	-	210
USGS-047	03/28/2022	2.5	U	3.4
USGS-052	03/29/2022	337	-	56
USGS-067	03/29/2022	128	-	23
USGS-123	03/02/2022	-1.7	UJ	3.4

Data qualifiers: U = undetected, J = estimate, R = rejected, "+" or "-" after a J means that the estimated result is biased high or low, respectively.
MDC range 5 – 10 pCi/L.

Table 18. Uranium isotope concentrations (pCi/L) for water samples, first quarter, 2022.

Sample Location	Sample Date	Uranium-234		Uranium-235		Uranium-238				
		Concentration	2 SD	Concentration	2 SD	Concentration	2 SD			
Aquifer Samples										
Facility										
<i>Test Area North</i>										
TAN-2336	01/11/2022	0.94	-	0.26	0.09	J#	0.074	0.173		0.098

Data qualifiers: U = undetected, J = estimate, R = rejected, "+" or "-" after a J means that the estimated result is biased high or low, respectively.
Result is >MDC and >2SD but <3SD and is therefore considered questionable and J-flagged as an estimate.
MDC (U-234) 0.09 pCi/L. MDC (U-235) 0.075 pCi/L. MDC (U-238) 0.087 pCi/L.

Table 19. Common ion concentrations (mg/L) in water samples, first quarter, 2022.

Sample Location	Sample Date	Calcium*	Magnesium*	Sodium*	Potassium*	Fluoride	Chloride	Sulfate	Alkalinity†	
Aquifer Samples										
Facility										
<i>Idaho Nuclear Technology and Engineering Center</i>										
ICPP-MON-A-230	03/28/2022	-	-	-	-	-	74.0 ¹	35.8	135	
USGS-047	03/28/2022	-	-	-	-	-	14.1	23.0	145	
USGS-052	03/29/2022	-	-	-	-	-	20.4	24.6	142	
USGS-067	03/29/2022	-	-	-	-	-	41.3 ¹	27.7	135	
USGS-123	03/02/2022	-	-	-	-	-	19.9	21.1	130	
<i>Test Area North</i>										
TAN-2336	01/11/2022	170	-	120	2600	23	125 ²	2.45 ²	UJ	5660

Data qualifiers: U = undetected, J = estimate, R = rejected, "<" = less than detection limit, "+" or "-" after a J means that the estimated result is biased high or low, respectively.

† As CaCO₃.

"-" = not analyzed.

Note 1. Lab indicated that a 5:1 dilution of this sample was required for this analyte.

Note 2. Lab indicated that a 20:1 dilution of this sample was required for this analyte.

Table 20. Dissolved metals concentrations (µg/L) in water samples, first quarter, 2022.

Sample Location	Sample Date	Arsenic	Barium	Chromium	Iron	Lead	Manganese	Selenium	Zinc								
Aquifer Samples																	
Facility																	
<i>Idaho Nuclear Technology and Engineering Center</i>																	
ICPP-MON-A-230	03/28/2022	-	-	-	5.7	-	-	-	-								
USGS-047	03/28/2022	-	-	-	7.4	-	-	-	-								
USGS-052	03/29/2022	-	-	-	7.0	-	-	-	-								
USGS-067	03/29/2022	-	-	-	6.7	-	-	-	-								
USGS-123	03/02/2022	-	-	-	6.2	-	-	-	-								
Test Area North																	
TAN-2336 ¹	01/11/2022	15 ²	UJ	1600	-	130	-	16000	-	0	U	2200	-	-	-	-	-

Samples were filtered in the field unless otherwise noted.

Data qualifiers: U = undetected, J = estimate, R = rejected, "<" = less than detection limit, "+" or "-" after a J means that the estimated result is biased high or low, respectively.

"-" = not analyzed.

Note 1. Lab indicated that a 12.5:1 dilution of this sample was required for these analytes.

Note 2. The MDC was elevated due to the 12.5:1 dilution and therefore this value is a non-detect.

Table 21. Dissolved nutrient concentrations (mg/L) in water samples, first quarter, 2022.

Sample Location	Sample Date	Nitrate + Nitrite*	Total Phosphorus
Aquifer Samples			
Facility			
<i>Idaho Nuclear Technology and Engineering Center</i>			
ICPP-MON-A-230	03/28/2022	6.8 ¹	-
USGS-047	03/28/2022	1.6	-
USGS-052	03/29/2022	2.1 ²	-
USGS-067	03/29/2022	5.2 ¹	-
USGS-123	03/02/2022	1.2 ³	-
Test Area North			
TAN-2336	01/11/2022	0.056	-

Samples were filtered in the field unless otherwise noted.

Data qualifiers: U = undetected, J = estimate, R = rejected, "<" = less than detection limit, "+" or "-" after a J means that the estimated result is biased high or low, respectively.

* As N.

"-" = not analyzed.

1. Lab indicated that a 5:1 dilution of this sample was required for this analyte.
2. Lab indicated that a 4:1 dilution of this sample was required for this analyte.
3. Lab indicated that a 2:1 dilution of this sample was required for this analyte.
4. Lab indicated that a 50:1 dilution of this sample was required for this analyte.

Table 22. Volatile organic compound concentrations (µg/L) in water samples, first quarter, 2022.

Sample Location	Sample Date	PCE		TCE		1,1-DCE		cis-1,2-DCE		trans-1,2-DCE		Vinyl chloride	
		<5.0	U	<5.0	U	<5.00	U	<5.0	U	<5.0	U	<5.0	U
TAN-2336 ¹	01/11/2022	<5.0	U	<5.0	U	<5.00	U	<5.0	U	<5.0	U	<5.0	U

Table 22. continued - Volatile organic compound concentrations (µg/L) in water samples, first quarter, 2022.

Sample Location	Sample Date	2-Butanone	Methylene Chloride	Chloro-methane	1,1-DCA				
TAN-2336 ¹	01/11/2022	1140 ²	-	21.2	J	<5.0	U	<5.0	U

Abbreviations: PCE = tetrachloroethene; TCE = trichloroethene; 1,1-DCE = 1,1-dichloroethene; cis-1,2-DCE = cis-1,2-dichloroethene; trans-1,2-DCE = trans-1,2-dichloroethene; 1,1-DCA = 1,1-dichloroethane. Data qualifiers: U = undetected, J = estimate, R = rejected, "<" = less than detection limit, "+" or "-" after a J means that the estimated result is biased high or low, respectively.

Note 1. Lab indicated that a 10:1 dilution of this sample was required for all analytes. This resulted in MDC increases by a factor of 10 from 0.5 to 5.0µg/L.

Note 2. Transient and expected to rapidly dissipate. Fowler, Thompson, and Mueller, 2011 (DOI: 10.1002/rem.21296).

Terrestrial Monitoring Results

The DEQ-INL OP conducts terrestrial (soil and milk) monitoring to characterize deposition and migration of contaminants and provide independent verification of DOE’s terrestrial monitoring programs. Physical soil sampling and *in-situ* gamma spectrometry are used to characterize actual deposition and accumulation of radioactive contaminants in soils. Milk samples are collected to evaluate the potential for ingestion of radioactivity by the population around the INL. No *in-situ* gamma spectrometric measurements were performed, nor were any soil samples physically collected during the first calendar quarter of 2022.

Milk

DEQ-INL OP monitors milk for the naturally occurring radionuclide potassium-40 (⁴⁰K) and man-made iodine-131 (¹³¹I). Milk samples are collected monthly. Results for gamma spectrometric analyses of milk samples are presented in **Table 23**. ⁴⁰K was detected in all samples within the expected range of concentrations. ¹³¹I was not detected. Based on measurements of radionuclides in milk, there were no discernable impacts to the off-site environment from INL operations.

Table 23. Gamma spectrometry analysis data for milk samples, first quarter, 2022.

Sample Location/Dairy	Sample Date	Naturally occurring Potassium-40		Man-made Iodine-131		
		Concentration ²	± 2 SD	Concentration ²	± 2 SD	MDC
Monitoring Samples						
Gooding	01/18/22	1412	127	0.5	2.4	4.0
	02/15/22	1443	88	1.1	2.7	4.4
	03/15/22	2004	109	-0.2	1.1	1.9
Verification Samples¹						
Minidoka	01/04/22	1568	93	0.3	1.7	2.9
Terreton	01/05/22	1363	112	0.5	1.9	3.1
Dietrich	02/01/22	1328	84	0.3	1.9	3.2
Rigby	02/01/22	1349	124	0.1	1.9	3.3
Minidoka	03/01/22	1423	116	0.4	1.2	2.0
Howe	03/03/22	1376	86	-0.7	2.3	3.8

¹ DEQ-INL OP samples collected by the off-site INL environmental surveillance contractor.

² Concentrations with associated uncertainties (± 2 SD) and minimum detectable concentrations (MDC) are expressed in pCi/L.

Quality Assurance

Measurements of constituent concentrations in environmental media are subject to inaccuracy from errors that may be introduced during the collection, transportation, and analysis of samples, calibration of equipment, and recording and reporting of results. While it is impossible to quantify every error that may affect a result, a quality assurance (QA) program can evaluate the overall quality of a dataset and, in many cases, identify and address errors or inaccuracies. DEQ-INL OP's QA program is designed to (1) ensure sample integrity, (2) evaluate the precision and accuracy of analytical results, and (3) ensure that the environmental data are representative and complete.

This section summarizes the quality assurance assessment of the data collected by DEQ-INL OP in the first quarter of 2022. Included are the results of quality control (QC) samples (blanks, duplicates, and spikes) that DEQ-INL OP submitted to Idaho State University's Environmental Monitoring Laboratory (ISU-EML) for radiological analyses and to the Idaho Bureau of Laboratories-Boise (IBL) for non-radiological analyses during the quarter. The analytical results of QC samples are used to assess the precision, accuracy, and representativeness of the environmental data presented in this report. During the first quarter of 2022, DEQ-INL OP submitted 74 QC samples for various radiological and non-radiological analyses (**Table 24**).

All samples referenced in this report were collected in accordance with written procedures maintained by the DEQ-INL OP. Analytical methods and QC procedures used by the laboratories were performed in accordance with approved written procedures maintained by each lab. QC samples analyzed by the labs as part of each lab's internal QA program are not discussed in this report.

Blank Samples

Blank samples consist of matrices that contain immeasurable or acceptably low concentrations of the analyte(s) of interest. They are used to monitor for contamination introduced during sample collection, storage, shipment, and analysis. For water matrices, a blank sample consists of 18-megohm deionized water from the DEQ-Idaho Falls Regional office and is categorized as a field blank, equipment blank, or trip blank depending on how the blank is handled. A field blank is used to monitor for contamination introduced from the environment during sample collection, an equipment blank is used to monitor for contamination introduced by contaminated equipment, and a trip blank is used to monitor for contamination introduced during transportation of samples (trip blanks are typically only used for VOCs). Most water blank samples submitted to laboratories by DEQ-INL OP are field blanks.

For all analyses except low-level tritium in water, a blank sample result is considered acceptable if it is less than or equal to the minimum detectable concentration (MDC). For low-level tritium analyses in water samples, a blank sample result is acceptable if it is less than or equal to 33 pCi/L.³ If a blank result exceeds acceptance criteria, above-MDC results in other samples collected, transported, or analyzed together with the failed blank may be qualified as biased high (J+) or rejected (R), or may remain unqualified, depending on the relative sizes of the blank detection and other sample results.

Sample results for blank TSP filters submitted for gross alpha and gross beta screening in air for the first quarter of 2022 are presented in **Table 25**. Blank sample results for select gamma emitters in air from TSP filters composited for the quarter are presented in **Table 26**. Data for blank analyses used to assess data quality for tritium in water vapor in air are presented in **Table 27**. Blank analysis results for 2021 TSP annual radiochemical composites of 4-inch filters are presented in **Table 28**.

³ The water used by DEQ-INL OP to create blank samples contains measurable concentrations of tritium produced cosmogenically and by above-ground testing of nuclear weapons during the twentieth century. The highest tritium concentration that DEQ considers acceptable in a blank is calculated as the mean tritium concentration in DEQ blanks from 2013 to 2017 plus two standard deviations (33 pCi/L).

The TSP blank 47-mm filter gross alpha result for the week of 2/16-2/23/22 minimally exceeded the MDC. The associated TSP gross alpha field results for that week were qualified as biased-high estimates (J+). All other blank sample results passed acceptance criteria in the first quarter of 2022.

Duplicate Samples

A duplicate sample is one that is collected at the same location and approximately the same time as another sample (referred to as the “original” sample). Duplicate sample results are compared to the original sample’s results to evaluate reproducibility. Significant differences between the two could indicate poor analytical precision or a non-uniform sample matrix.

The difference between the results of an original and duplicate sample (referred to below as a “duplicate-sample pair”) is evaluated differently for radiological and non-radiological analyses. For radiological analyses, the results of a duplicate-sample pair are considered to be in agreement if their absolute difference is less than or equal to three times the pooled error of the results:

$$|R_1 - R_2| \leq 3\sqrt{S_1^2 + S_2^2}$$

R_1 = Original sample result

R_2 = Duplicate sample result

S_1 = Analytical uncertainty (1 SD) of the original result

S_2 = Analytical uncertainty (1 SD) of the duplicate result

Radiological results are also considered to be in agreement if their relative percent difference (RPD) is no more than ± 20 percent. RPD is calculated as:

$$RPD = \frac{R_1 - R_2}{(R_1 + R_2)/2} \times 100$$

For non-radiological analyses, the RPD is used to evaluate duplicate sample pairs in which both results exceed five times the MDC. An RPD of up to ± 20 percent is acceptable. If one or both sample results is less than five times the MDC, the results agree if their absolute difference is less than or equal to the MDC.

Duplicate results for radiological analyses in groundwater and surface water are presented in **Table 29**. Duplicate results for metals, common ions and nutrients, and VOCs in groundwater are presented in **Tables 30, 31, and 32**.

The gross alpha duplicate results for USGS-052 failed acceptance criteria. The other duplicate gross alpha results were acceptable. No sample results were qualified as a result of this failure. The technetium-99 (Tc-99) duplicate results for USGS-123 failed acceptance criteria. The original USGS-123 Tc-99 result was qualified as an estimate (J-flag) due to its large disagreement with the duplicate result. There were no other field sample Tc-99 results associated with this duplicate. The other duplicate technetium-99 results were acceptable. The methylene chloride duplicate results for TAN-2336 failed acceptance criteria. The original TAN-2336 result was flagged as an estimate. There were no other field sample VOC results associated with this duplicate. All other duplicate results passed acceptance criteria in the first quarter of 2022.

Spiked Samples

Spiked samples are samples to which known concentrations of specific analytes have been added. They are used to assess a laboratory’s analytical accuracy. The percent recovery (%R) of each spiked-sample analysis is calculated as the ratio of the spike concentration determined by the lab to the known spike concentration. DEQ-INL OP considers the lab’s result to be in control if the percent recovery is $100 \pm$

25%. If the percent recovery of a spiked sample is 50-74%, above-MDC results of samples analyzed in the same batch as the spiked sample may be qualified as low-biased estimates (J-), and below-MDC results may be qualified as undetected estimates (UJ). If the percent recovery of a spiked sample is 126-150%, above-MDC results of associated samples may be qualified as high-biased estimates (J+), and below-MDC results may be qualified as undetected (U). If the percent recovery of a spiked sample is <50% or >150%, the results of all associated samples may be qualified as rejected (R), except for sample results below MDC associated with a spiked-sample analysis having a percent recovery >150%, in which case the sample result remains qualified as undetected (U).

No spiked water samples were analyzed during the first quarter of 2022.

DEQ-INL OP also prepares additional “spike-like” quality control samples to assess ambient radiation measurement bias. Once per quarter, DEQ-INL OP irradiates several electret ionization chambers (EICs) to verify EIC response. Irradiations of EICs are conducted in a repeatable geometry to a known exposure of near 30 mR and two additional higher and lower exposures, ranging from 15 to 60 mR. EIC responses are compared directly with the exposure received from the NIST traceable cesium-137 source provided by ISU-EML. EIC response is considered acceptable if each measurement has a percent recovery of $100 \pm 25\%$ when compared to the known irradiated quantity. Overall response for each control set is considered acceptable if the average of the three individual results for the set has a percent recovery of $100 \pm 25\%$. The irradiation results for first quarter 2022 are presented in **Table 31**. Real-time pressure correction is used to calculate the net exposure measured by these EIC control sets. All individual EIC measurements and average EIC results passed DEQ-INL OP acceptance criteria.

Laboratory QC Issues

Radiochemical analyses results for composited 4-inch TSP filters from calendar year 2021 are presented in **Table 8** of this report. The ISU vendor lab (ALS, Fort Collins, CO.) non-conformance report listed significant quality control issues associated with the analyses of these samples: **(1)** The report stated the chemical yields for all samples are below the 30% lower control limit for Pu-238 and Pu-239/240, ranging from 1.45% to 20.5%. Additionally, eight of thirteen samples for Am-241 analysis have yields below the 30% lower control limit, ranging from 1.79% to 23.9%. The chemical carrier recoveries for Sr-90 for all samples are below the 40% lower control limit ranging from 10.3% to 22.6%. The vendor lab stated that spectral quality is adequate for accurate quantification. The lab believes that the low yields and recoveries are caused by matrix interference, as another client’s samples prepared in the batch with these samples had acceptable yields, as did the batch method blank, laboratory control sample, and laboratory control sample duplicate. The samples all had large amounts of iron in them, as seen in Inductively Coupled Plasma (ICP) data. The samples were all counted for 1000 minutes to achieve the lowest MDC possible. **(2)** The Quality Control section of the report indicated that Pu-239/240 activity in the associated method blank was above the MDC value. However, the measured blank activity was below the requested MDC. **(3)** The vendor lab analysts who ashed these samples stated that the filter samples were much larger and thicker than they are used to, so they did not completely ash. Additionally, the small muffle furnace that goes up to 600 degrees Celsius was out of service, so they could only ash in the larger ovens that only go up to 450 degrees Celsius. This caused the aqueous samples to be much darker than normal after ashing and the three-acid digestion. It is believed that this caused significant interference when the samples were loaded onto the chloride ion-exchange columns for isotopic Pu and Am-241 separation, and then the strontium column. Because of these non-conformances, all 2021 4-inch TSP filter composite radiochemical analysis results are J-flagged as estimates. The validity of these results is in doubt and their accuracy is questionable.

DEQ-INL OP Equipment QC Issue

Many of the 47-mm filter TSP air samplers were found to be operating outside of their expected flow rate range in the first quarter, 2022. A calibrated flow rate gauge was taken into the field weekly to measure the sample start and stop flow rates of these samplers. The calibrated flow rate gauge measurements were used for the activity concentration calculations in the suspect measurements, and results are considered (usable) estimates.

Qualification of Low Level Sample Results

Sample results >MDC are generally considered detections, with the following exceptions⁴ that apply primarily to radionuclide concentrations in water samples:

1. Results >MDC but $\leq 2SD$ are considered non-detections and U-flagged as undetected, where SD is the sample standard deviation.
2. Results >MDC and >2SD but <3SD are considered questionable detections and J-flagged as estimates.

Analytical QA/QC Assessment

Other than those discussed above, no issues involving sample chain of custody, sample holding times, and the analysis of blank, duplicate, and spiked samples were observed during the first quarter of 2022 which significantly affected data quality. The ratio of total QC analyses to total field sample analyses of 14.1% is acceptable and above the DEQ-INL OP minimum requirement of 10%. Methodologies and data reports issued by the contracting laboratories generally conformed to the requirements of DEQ-INL OP during the first quarter of 2022.

Data usability is the measure of field sample results that are not rejected divided by the total number of field sample results obtained. The overall data usability (non-rejected results divided by the total number of field sample results reported) of 99.6% for the first quarter of 2022 is excellent and well above the acceptable value of 90% for the DEQ-INL OP ESP and is summarized in **Table 24**. The overall data completeness (usable results divided by the total number of field sample results expected) of 98.5% is also well above the acceptable value of 90%.

Preventative Maintenance and Equipment Reliability

All equipment was calibrated and checked according to prescribed periodicity. The Experimental Field Station 47-mm filter TSP and radioiodine sampler was out of service from 2/16/22 to 3/04/22 for motor maintenance/replacement. The Howe and Mud Lake 8x10-inch filter TSP samplers were not deployed until 1/12/22. The Experimental Field Station and Idaho Falls Duplicate 8x10-inch filter TSP samplers were not deployed until 1/26/22. The Fort Hall 8x10-inch filter TSP sampler was not started up until 2/15/22. Service reliability for air sampling equipment for the first quarter of 2022 is summarized in **Table 34**.

Conclusion

All data collected for the first quarter of 2022 have been assigned the applicable qualifiers to designate the appropriate use of the data. The overall data usability of 99.6% and data completeness of 98.5% are well above the acceptable value of 90% for the quarter, with the data meeting the requirements and data quality objectives established by DEQ-INL OP.

⁴ Monitoring and Surveillance Committee, Consistency in Reporting Results Subcommittee Meeting Summary, 2/5/04 and 4/1/04.

Table 24. Summary of the analyses performed, first quarter, 2022.

Media Sampled	Collection Device	Analyte	Sample Analyses	Blank Analyses	Duplicate Analyses	Spike Analyses	Data Rejected ¹	Analyzing Lab ²
Air								
Particulate	4-inch filter	Gross alpha	140	13	0	0	1	ISU-EML
		Gross beta	140	13	0	0	1	ISU-EML
		Gamma emitters	44	4	0	0	0	ISU-EML
		Radiochemical	12	1	0	0	0	ISU Sub
Water Vapor	Desiccant column	Tritium	21	6	0	0	0	ISU-EML
Gaseous	Charcoal filter	Iodine-131	13	0	0	0	0	ISU-EML
Precipitation	Poly bottle	Tritium	6	0	0	0	0	ISU-EML
		Gamma emitters	6	0	0	0	0	ISU-EML
Water								
Groundwater & Surface Water	Grab or composite	Gross alpha	6	0	3	0	0	ISU-EML
		Gross beta	6	0	3	0	0	ISU-EML
		Gamma emitters	6	0	3	0	0	ISU-EML
		Tritium	6	0	3	0	0	ISU-EML
		Low-level tritium	0	0	0	0	0	ISU-EML
		Radiochemical ⁶	12	0	6	0	0	ISU Sub
		Metals	6	0	3	0	0	IBL
		Common Ions	6	0	3	0	0	IBL
		Nutrients	6	0	3	0	0	IBL
Volatile Organics	1	0	1	0	0	IBL		
Terrestrial								
Milk	Grab or composite	Gamma emitters	9	0	0	0	0	ISU-EML
Soil	<i>in situ</i>	Gamma emitters	0	0	0	0	0	DEQ-INL OP
	Grab – “puck”	Gamma emitters	0	0	0	0	0	ISU-EML
Radiation								
Ambient	EICs	Gamma Radiation	67	0	0	9	0	DEQ-INL OP
	EcoGamma	Gamma Radiation	11	NA	NA	NA	0	DEQ-INL OP
Total analyses performed			524	37	28	9	2	
Total QC analyses performed (blanks, duplicates, and spikes)			74					
Ratio of total QC analyses to total sample analyses³			14.1%					
Data usability⁴, percent			99.6%					
Data completeness⁵, percent			98.5%					

¹ Combined Laboratory and DEQ-INL OP rejection criteria (data was rejected for any reason).

² ISU-EML = Idaho State University – Environmental Monitoring Laboratory; ISU Sub = Subcontract laboratory to ISU-EML; IBL = Idaho Bureau of Laboratories, Boise; IBL Sub = Subcontract laboratory to IBL; DEQ-INL OP = Analyzed by INL Oversight Program, Idaho Department of Environmental Quality.

³ DEQ-INL OP requires that the number of QC analyses performed be at least 10 percent of the number of sample analyses performed.

⁴ Data usability is calculated as [total analyses – rejected data]/[total analyses]. DEQ-INL OP considers a data usability rate of 90 percent or higher to be acceptable.

⁵ Data completeness is calculated as usable results divided by the total number of field sample results expected. DEQ-INL OP considers a data completeness rate of 90 percent or higher to be acceptable.

⁶ Radiochemical includes Strontium-90, Technetium-99, Uranium 234, 235, and 238, Plutonium-238, 239/240, and Americium-241.

Table 25. Blank analysis results for gross alpha and beta in 47-mm particulate air (TSP) filters, first quarter, 2022.

Collection Period		Corrected volume (m ³) ¹	Gross alpha			Gross beta		
Start	Stop		Value	± 2 SD	MDC	Value	±2 SD	MDC
12/29/21	01/05/22	525	-0.2	0.3	0.6	0.3	0.6	1.0
01/05/22	01/12/22	525	0.0	0.3	0.6	-0.1	0.6	1.0
01/12/22	01/19/22	525	-0.1	0.3	0.6	-0.8	0.7	1.2
01/19/22	01/26/22	525	-0.1	0.3	0.7	-0.7	0.7	1.2
01/26/22	02/02/22	525	0.0	0.3	0.6	-0.5	0.7	1.2
02/02/22	02/09/22	525	0.1	0.3	0.6	-0.3	0.6	1.0
02/09/22	02/16/22	525	-0.1	0.3	0.5	-1.1	0.6	1.2
02/16/22	02/23/22	525	0.5	0.3	0.4	0.2	0.8	1.3
02/23/22	03/02/22	525	0.0	0.3	0.6	0.2	0.8	1.3
03/02/22	03/09/22	525	-0.1	0.3	0.6	0.3	0.7	1.2
03/09/22	03/16/22	525	0.0	0.3	0.6	-0.7	0.7	1.3
03/16/22	03/23/22	525	0.1	0.4	0.6	-0.6	0.7	1.2
03/23/22	03/30/22	525	-0.1	0.3	0.7	-0.8	0.7	1.3

Note: Concentrations, associated uncertainties (± 2 SD), and minimum detectable concentrations (MDC) are expressed in 1×10^{-3} pCi/m³.

¹ A volume equal to the average of the weekly volumes collected through each valid field filter was used to compute “concentrations” for the blank for meaningful comparison to sample results. No air was passed through the blank filters.

Table 26. Blank analysis results for gamma spectrometry for 47-mm TSP air filters, quarterly composite samples, first quarter, 2022.

Analysis Date	Beryllium-7			Ruthenium-106/Rhodium-106			Antimony-125		
	Concentration ¹	± 2 SD	MDC	Concentration	± 2 SD	MDC	Concentration	± 2 SD	MDC
04/26/2022	-41	82	147	-17	132	237	-5	14	24
Analysis Date	Cesium-134			Cesium-137					
	Concentration ¹	± 2 SD	MDC	Concentration	± 2 SD	MDC	Concentration	± 2 SD	MDC
04/26/2022	-62	9	15	-20	9	16			

Note: Concentrations are expressed in 1×10^{-5} pCi/m³ with associated uncertainty (± 2 SD) and minimum detectable concentration (MDC).

¹ These concentrations are from blank filters collected weekly, composited, and analyzed for the calendar quarter. A composite volume equal to the sum of the weekly average volumes collected through each valid field filter was used to compute “air concentrations” for the blank for meaningful comparison to sample results. No air was passed through the blank filters.

Table 26 (continued). Blank analysis results for gamma spectrometry for 8x10-inch TSP air filters, monthly composite samples, first quarter, 2022.

Month	Beryllium-7			Ruthenium-106/Rhodium-106			Antimony-125		
	Concentration ¹	± 2 SD	MDC	Concentration	± 2 SD	MDC	Concentration	± 2 SD	MDC
Jan 2022	-5	19	34	0	68	118	-6	5	10
Feb 2022	-1	22	38	0	68	118	3	7	12
Mar 2022	-11	25	44	-38	54	95	-2	5	9
Month	Cesium-134			Cesium-137					
	Concentration ¹	± 2 SD	MDC	Concentration	± 2 SD	MDC	Concentration	± 2 SD	MDC
Jan 2022	1	2	4	1	2	4			
Feb 2022	-2	2	4	-1	3	5			
Mar 2022	0	2	3	0	2	4			

Note: Concentrations are expressed in 1×10^{-5} pCi/m³ with associated uncertainty (± 2 SD) and minimum detectable concentration (MDC).

¹ These concentrations are from blank filters collected weekly, composited, and analyzed for the calendar month. A composite volume equal to the sum of the weekly average volumes collected through each valid field filter was used to compute “air concentrations” for the blank for meaningful comparison to sample results. No air was passed through the blank filters.

Table 27. Blank analysis results for 2021 TSP annual radiochemical composites of 4-inch air filters.

Sample Description	⁹⁰ Sr			²³⁸ Pu			²³⁹ Pu/ ²⁴⁰ Pu			²⁴¹ Am		
	Value ¹	± 2 SD	MDC	Value ¹	± 2 SD	MDC	Value ¹	± 2 SD	MDC	Value ¹	± 2 SD	MDC
Blank	1.57	3.52	5.84	-0.03	0.16	0.30	0.00	0.16	0.30	0.01	0.08	0.15

Note: Concentrations are expressed in 1×10^{-5} pCi/m³ with associated uncertainty (± 2 SD) and minimum detectable concentration (MDC).

¹ These concentrations are from blank filters collected weekly, composited, and analyzed for the calendar year. A composite volume equal to the sum of the weekly average volumes collected through each valid field filter was used to compute “air concentrations” for the blank for meaningful comparison to sample results. No air was passed through the blank filters.

Table 28. Blank analysis results for tritium in water vapor from air samples, first quarter, 2022.

Sample Number	Start Date	Collection Date	Analysis Date	Tritium		
				Concentration	± 2 SD	MDC
OP221ZTR01	03/08/2022	03/11/2022	03/16/2022	-0.02	0.10	0.17
OP221ZTR02	03/08/2022	03/11/2022	03/16/2022	0.01	0.10	0.17
OP221ZTR03	03/23/2022	03/31/2022	04/11/2022	0.02	0.09	0.16
OP221ZTR04	03/23/2022	03/31/2022	04/11/2022	-0.07	0.09	0.16
OP221 FRIDGE	03/08/2022	04/12/2022	04/15/2022	-0.01	0.10	0.17
OP221 SINK	03/08/2022	04/12/2022	04/15/2022	0.02	0.10	0.17

Note: Concentrations are expressed in nCi/L with associated uncertainty (± 2 SD) and minimum detectable concentration (MDC).

Table 29. Duplicate sample results (pCi/L) for radiological constituents in groundwater and/or surface water, first quarter, 2022.

Analysis/Sample Location	Original Sample Number	Concentration	± 2 SD	Duplicate Sample Number	Concentration	± 2 SD	RPD	R ₁ -R ₂	3(S ₁ ² +S ₂ ²) ^{1/2}	Within Criteria?
Gross Alpha										
TAN-2336	221W001	2.7	24.8	221W009	-18.8	22.1	-267	21.5	49.8	Yes
USGS-123	221W017	2.0	0.7	221W025	3.2	0.9	-46	1.2	1.7	Yes
USGS-052	221W033	-0.5	0.8	221W055	2.9	0.9	-283	3.4	1.8	No
Gross Beta										
TAN-2336	221W001	1601.5	48.9	221W009	1654.3	49.4	-3	52.8	104.3	Yes
USGS-123	221W017	2.5	0.9	221W025	1.5	0.9	50	1.0	1.9	Yes
USGS-052	221W033	246.2	3.5	221W055	242.2	3.6	2	4.0	7.5	Yes
Cesium-137										
TAN-2336	221W001	6.1	1.9	221W009	3.8	1.9	46	2.3	4.0	Yes
USGS-123	221W017	-0.4	1.1	221W025	1.2	1.4	-400	1.6	2.7	Yes
USGS-052	221W033	0.3	1.3	221W055	-0.6	1.7	-600	0.9	3.2	Yes
Tritium (standard method)										
TAN-2336	221W003	480	110	221W011	530	110	-10	50	233	Yes
USGS-123	221W021	1550	160	221W029	1540	160	1	10	339	Yes
USGS-052	221W036	310	120	221W058	510	120	-49	200	255	Yes
Strontium-90										
TAN-2336	221W002	590	140	221W010	540	130	9	50	287	Yes
USGS-123	221W019	0.50	0.38	221W027	0.13	0.35	117	0.37	0.77	Yes
USGS-052	221W034	1.93	0.54	221W056	1.77	0.51	9	0.16	1.11	Yes
Uranium-234										
TAN-2336	221W004	0.94	0.26	221W012	0.85	0.29	10	0.09	0.58	Yes
Uranium-235										
TAN-2336	221W004	0.09	0.074	221W012	0.042	0.081	73	0.048	0.165	Yes
Uranium-238										
TAN-2336	221W004	0.173	0.098	221W012	0.27	0.15	-44	0.097	0.269	Yes
Technetium-99										
USGS-123	221W020	-1.7	3.4	221W028	12.8	5.5	-261	14.5	8.5	No
USGS-052	221W035	337	56	221W057	401	67	-17	64	131	Yes

RPD = relative percent difference.

Table 30. Duplicate results for metals (µg/L) in groundwater, first quarter, 2022.

Sample Location	Sample Number	Sample Date	Arsenic	Barium	Chromium	Iron	Lead	Manganese	Selenium	Zinc
TAN-2336 ¹	221W006	1/11/2022	15 UJ	1600	130	16000	<12	2200	-	-
TAN-2336 ¹	221W014	1/11/2022	16 UJ	1600	140	15000	<12	2300	-	-
RPD			-6	0	-7	6	0	-4	-	-
USGS-123	221W023	3/02/2022	-	-	6.2	-	-	-	-	-
USGS-123	221W031	3/02/2022	-	-	6.4	-	-	-	-	-
RPD			-	-	-3	-	-	-	-	-
USGS-052	221W038	3/29/2022	-	-	7.0	-	-	-	-	-
USGS-052	221W060	3/29/2022	-	-	7.2	-	-	-	-	-
RPD			-	-	-3	-	-	-	-	-

RPD = relative percent difference.

Data qualifier: J = estimate.

¹ Lab indicated that a 12.5:1 dilution of this sample was required for these analytes. The elevated MDCs indicate that the arsenic values are non-detections.

Table 31. Duplicate sample results for common ions and nutrients (mg/L) in groundwater, first quarter, 2022.

Sample Location	Sample Number	Sample Date	Calcium	Magnesium	Sodium	Potassium	Fluoride	Chloride	Sulfate	Total Alkalinity [†]	Total Nitrogen	Total Phosphorus
TAN-2336	221W001 ¹ , 006, 007 ⁵	1/11/2022	170	120	2600	23	-	125 J ²	2.45 J ²	5560	0.056	11
TAN-2336	221W013 ¹ , 014, 015 ⁵	1/11/2022	160	120	2500	22	-	129 J ²	2.24 J ²	5580	0.064	12
RPD			6	0	4	4	-	-3	9	-0.4	-13	-9
USGS-123	221W022, 024 ⁴	3/02/2022	-	-	-	-	-	19.9	21.1	130	1.2	-
USGS-123	221W030, 032	3/02/2022	-	-	-	-	-	19.8	21.1	130	1.0	-
RPD			-	-	-	-	-	0.5	0	0	18	-
USGS-052	221W037, 039 ³	3/29/2022	-	-	-	-	-	20.4	24.6	142	2.1	-
USGS-052	221W059, 061 ⁴	3/29/2022	-	-	-	-	-	20.6	24.7	142	2.1	-
RPD			-	-	-	-	-	-1	-0.4	0	0	-

RPD = relative percent difference.

[†] As CaCO₃.

¹Sample required a 20:1 dilution for chloride and sulfate analyses.

²Results are flagged as estimates. The matrix spike and matrix spike duplicate did not meet the laboratory's acceptance criteria.

³Sample required a 4:1 dilution for total nitrogen analysis.

⁴Sample required a 2:1 dilution for total nitrogen analysis.

⁵Sample required a 50:1 dilution for total phosphorus analysis.

Table 32. Duplicate sample results (µg/L) for VOCs in water, first quarter, 2022.

	Sample Number	Sample Date	PCE	TCE	1,1-DCE	cis-1,2-DCE	trans-1,2-DCE	Vinyl Chloride	1,1-DCA	Carbon Tetrachloride	Methylene Chloride	Chloro-methane	Styrene	Chloro-form	MEK
TAN-2336	221W008 ¹	1/11/22	<5	<5	<5	<5	<5	<5	<5	<5	21.2	<5	<5	<5	1140 J ²
TAN-2336	221W016 ¹	1/11/22	<5	<5	<5	<5	<5	<5	<5	<5	14.7	<5	<5	<5	1170 J ²
RPD			0	0	0	0	0	0	0	0	36	0	0	0	-3

RPD = relative percent difference.

Abbreviations: PCE = tetrachloroethene; TCE = trichloroethene; 1,1-DCE = 1,1-dichloroethene; cis-1,2-DCE = cis-1,2-dichloroethene; trans-1,2-DCE = trans-1,2-dichloroethene; 1,1-DCA = 1,1-dichloroethane, MEK = Methyl Ethyl Ketone (2-Butanone).

¹Sample required a 10:1 dilution.

²Results are flagged as estimates. The matrix spike and matrix spike duplicate did not meet the laboratory's acceptance criteria.

Table 33. Electret ionization chamber (EIC) irradiation results (categorized as spiked samples), first quarter, 2022.

Electret #	Exposure Received		Net Measured Exposure ¹		%R	Within Spec?
	(mR)	Uncertainty (±1 SD, mR)	(mR)	Uncertainty (±1 SD, mR)		
SKR267	40.0	2.0	32.2	1.3	80.6%	Yes
SKR342	40.0	2.0	36.7	1.4	91.9%	Yes
SJX015	40.0	2.0	31.9	1.3	79.7%	Yes
Triplicate AVG:					84.1	Yes
SKR589	30.0	1.5	26.6	1.4	88.7%	Yes
SJX031	30.0	1.5	22.9	1.3	76.3%	Yes
SKR401	30.0	1.5	24.8	1.3	82.7%	Yes
Triplicate AVG:					82.6%	Yes
SKR353	20.0	1.0	16.8	1.3	84.0%	Yes
SJX054	20.0	1.0	17.1	1.3	85.5%	Yes
SKR496	20.0	1.0	16.8	1.4	84.0%	Yes
Triplicate AVG:					84.5%	Yes

Note: A percent recovery (%R) of 100 ± 25 is considered acceptable.

¹ Net measured exposure estimate includes a correction for atmospheric pressure.

Table 34. Air sampling field equipment service reliability (percent operational), first quarter, 2022.

Station Locations	Sample Type				
	47-mm TSP	8x10-inch TSP	Radioiodine	Atmospheric Moisture	Precipitation
Onsite Locations					
Big Lost River Rest Area	100%	100%	100%	100%	100%
Experimental Field Station	85%	100% ³	85%	100%	NC ¹
Sand Dunes Tower	100%	100%	100%	100%	NC ¹
Van Buren Avenue	100%	100%	100%	100%	NC ¹
Boundary Locations					
Atomic City	100%	100%	100%	100%	100%
Howe	100%	100% ⁴	100%	100%	100%
Montevue	100%	100%	100%	100%	100%
Mud Lake	100%	100% ⁴	100%	100%	100%
Distant Locations²					
Craters of the Moon	100%	100%	100%	100%	NC ¹
Idaho Falls	100%	100%	100%	100%	100%
Idaho Falls Duplicate ⁵	NC ¹	100% ³	-	-	-

Note: The values in this table were calculated by dividing the number of weeks the equipment was in operation by the number of weeks in the quarter.

¹ NC = Sample not collected at this location.

² The Fort Hall Station, operated by the Shoshone-Bannock Tribes, is not included here.

³ The Experimental Field Station and Idaho Falls Duplicate 8x10-inch samplers were 100% operational from their 1/26/22 deployment to the end of the quarter.

⁴ The Howe and Mud Lake 8x10-inch samplers were 100% operational from their 1/12/22 deployment to the end of the quarter.

⁵ A duplicate 8x10-inch filter TSP sampler is currently installed at the Idaho Falls location.

Appendix A

Table A-1. Weekly concentrations (in 1×10^{-3} pCi/m³) for gross alpha and gross beta analyses of 47-mm TSP filters for all locations, first quarter, 2022.

Sample Location	Collection Date		Gross Alpha		Gross Beta	
	Start	Stop	Concentration	±2 SD	Concentration	±2 SD
On-Site Locations						
Big Lost River Rest Area	12/29/21	01/05/22	0.4	0.5	20.6	1.6
	01/05/22	01/12/22	0.4	0.5	20.4	1.6
	01/12/22	01/19/22	0.9	0.6	52.5	2.5
	01/19/22	01/26/22	0.5	0.6	33.4	2.1
	01/26/22	02/02/22	0.3	0.5	34.5	2.1
	02/02/22	02/09/22	0.2	0.5	34.5	2.0
	02/09/22	02/16/22	0.7	0.5	40.8	2.2
	02/16/22	02/23/22	0.9 J+ ⁹	0.5 J+ ⁹	23.4	1.9
	02/23/22	03/02/22	0.8	0.6	45.7	2.4
	03/02/22	03/09/22	0.2	0.5	28.8	2.0
	03/09/22	03/16/22	0.3	0.5	15.6	1.7
	03/16/22	03/23/22	0.4	0.6	16.1	1.8
	03/23/22	03/30/22	0.7	0.5	23.1	1.5
	Experimental Field Station	12/29/21	01/05/22	0.3	0.4	26.2
01/05/22		01/12/22 ¹	0.4	0.4	27.6	1.5
01/12/22		01/19/22 ¹	0.8	0.4	59.4	2.1
01/19/22		01/26/22 ¹	0.6	0.4	39.8	1.8
01/26/22		02/02/22 ¹	0.4	0.4	42.7	1.8
02/02/22		02/09/22	0.6	0.3	39.9	1.6
02/09/22		02/16/22 ²	R	R	R	R
02/16/22		02/23/22 ³	NS	NS	NS	NS
02/23/22		03/02/22 ³	NS	NS	NS	NS
03/04/22		03/09/22 ⁴	0.1	0.4	19.7	1.6
03/09/22		03/16/22	0.2	0.3	17.4	1.2
03/16/22		03/23/22	0.8	0.4	16.9	1.2
03/23/22		03/30/22	1.0	0.5	25.9	1.5
Sand Dunes Tower	12/29/21	01/05/22	0.4	0.3	27.8	1.4
	01/05/22	01/12/22	0.1	0.3	25.1	1.4
	01/12/22	01/19/22	0.8	0.4	66.0	2.2
	01/19/22	01/26/22	0.6	0.4	38.3	1.7
	01/26/22	02/02/22 ¹	0.6	0.4	57.2	2.2
	02/02/22	02/09/22	0.3	0.3	37.7	1.6
	02/09/22	02/16/22 ¹	1.1	0.4	49.0	1.8
	02/16/22	02/23/22 ⁵	NS	NS	NS	NS
	02/23/22	03/02/22	0.8	0.4	44.3	1.8
	03/02/22	03/09/22	0.6	0.4	30.4	1.6
	03/09/22	03/16/22	0.2	0.4	13.9	1.3
	03/16/22	03/23/22 ¹	0.4	0.4	16.1	1.2
	03/23/22	03/30/22 ¹	0.4	0.4	26.0	1.5

Table A-1 continued. Weekly concentrations (in 1×10^{-3} pCi/m³) for gross alpha and gross beta analyses of 47-mm TSP filters for all locations, first quarter, 2022.

Sample Location	Collection Date		Gross Alpha		Gross Beta	
	Start	Stop	Concentration	± 2 SD	Concentration	± 2 SD
Van Buren Avenue	12/29/21	01/05/22	0.4	0.3	19.9	1.2
	01/05/22	01/12/22	0.1	0.3	23.3	1.3
	01/12/22	01/19/22	0.6	0.4	48.6	1.8
	01/19/22	01/26/22	0.5	0.4	35.1	1.6
	01/26/22	02/02/22	0.7	0.4	32.0	1.5
	02/02/22	02/09/22	0.6	0.4	34.5	1.6
	02/09/22	02/16/22	0.8	0.4	46.8	1.9
	02/16/22	02/23/22	0.9 J+ ⁹	0.3 J+ ⁹	27.2	1.5
	02/23/22	03/02/22	0.6	0.4	44.8	1.9
	03/02/22	03/09/22	0.6	0.4	29.3	1.6
	03/09/22	03/16/22 ¹	0.2	0.3	15.9	1.2
	03/16/22	03/23/22	0.3	0.4	17.4	1.3
	03/23/22	03/30/22 ¹	0.2	0.4	25.4	1.5
Boundary Locations						
Atomic City	12/29/21	01/05/22	0.6	0.4	20.3	1.2
	01/05/22	01/12/22	0.6	0.4	24.4	1.3
	01/12/22	01/19/22	0.7	0.4	65.8	2.1
	01/19/22	01/26/22	0.7	0.4	33.4	1.5
	01/26/22	02/02/22	0.6	0.3	39.6	1.6
	02/02/22	02/09/22	0.5	0.4	39.3	1.7
	02/09/22	02/16/22	1.0	0.4	36.6	1.6
	02/16/22	02/23/22 ¹	1.0 J+ ⁹	0.4 J+ ⁹	28.5	1.6
	02/23/22	03/02/22 ¹	0.9	0.4	52.5	2.1
	03/02/22	03/09/22	0.3	0.3	23.6	1.3
	03/09/22	03/16/22	0.1	0.3	16.0	1.2
	03/16/22	03/23/22	0.7	0.4	17.3	1.2
	03/23/22	03/30/22	0.8	0.4	23.3	1.4
Howe	12/29/21	01/05/22 ¹	0.4	0.4	26.2	1.4
	01/05/22	01/12/22 ¹	0.2	0.4	20.6	1.3
	01/12/22	01/19/22 ¹	0.9	0.4	55.5	2.0
	01/19/22	01/26/22 ¹	0.8	0.4	37.7	1.7
	01/26/22	02/02/22 ¹	0.8	0.4	48.1	1.9
	02/02/22	02/09/22 ⁶	0.5	0.4	35.7	1.7
	02/09/22	02/16/22 ¹	1.1	0.4	55.6	2.1
	02/16/22	02/23/22 ¹	0.9 J+ ⁹	0.3 J+ ⁹	31.9	1.7
	02/23/22	03/02/22 ¹	1.1	0.5	49.6	2.0
	03/02/22	03/09/22	0.4	0.4	33.7	1.8
	03/09/22	03/16/22	0.2	0.3	18.2	1.3
	03/16/22	03/23/22	0.6	0.4	17.3	1.3
	03/23/22	03/30/22	0.4	0.4	23.9	1.5

Table A-1 continued. Weekly concentrations (in 1×10^{-3} pCi/m³) for gross alpha and gross beta analyses of 47-mm TSP filters for all locations, first quarter, 2022.

Sample Location	Collection Date		Gross Alpha		Gross Beta	
	Start	Stop	Concentration	±2 SD	Concentration	±2 SD
Montevieu	12/29/21	01/05/22	0.4	0.3	25.9	1.4
	01/05/22	01/12/22	0.3	0.3	22.6	1.3
	01/12/22	01/19/22	0.9	0.4	63.1	2.1
	01/19/22	01/26/22	0.4	0.4	28.2	1.5
	01/26/22	02/02/22	0.5	0.4	42.0	1.7
	02/02/22	02/09/22	0.5	0.4	32.9	1.5
	02/09/22	02/16/22	1.2	0.4	45.4	1.8
	02/16/22	02/23/22	0.9 J+ ⁹	0.3 J+ ⁹	30.2	1.6
	02/23/22	03/02/22	1.1	0.4	47.4	1.9
	03/02/22	03/09/22	0.3	0.3	24.0	1.4
	03/09/22	03/16/22	0.3	0.4	16.9	1.3
	03/16/22	03/23/22	0.2	0.4	15.7	1.3
	03/23/22	03/30/22	0.6	0.4	23.1	1.4
Mud Lake	12/29/21	01/05/22	0.4	0.3	29.1	1.4
	01/05/22	01/12/22	0.3	0.3	22.3	1.3
	01/12/22	01/19/22	1.0	0.4	61.2	2.0
	01/19/22	01/26/22	0.5	0.4	30.1	1.5
	01/26/22	02/02/22	0.4	0.4	50.1	1.9
	02/02/22	02/09/22	0.6	0.4	33.8	1.5
	02/09/22	02/16/22	1.0	0.4	43.9	1.8
	02/16/22	02/23/22	0.8 J+ ⁹	0.3 J+ ⁹	27.9	1.5
	02/23/22	03/02/22	1.3	0.4	46.6	1.9
	03/02/22	03/09/22	0.5	0.4	28.7	1.5
	03/09/22	03/16/22	0.5	0.4	17.4	1.3
	03/16/22	03/23/22 ¹	0.4	0.4	18.5	1.4
	03/23/22	03/30/22 ¹	0.9	0.4	25.6	1.5
Distant Locations						
Craters of the Moon	12/29/21	01/05/22	0.0	0.4	15.8	1.4
	01/05/22	01/12/22	0.2	0.5	13.2	1.4
	01/12/22	01/19/22	0.1	0.5	32.2	2.0
	01/19/22	01/26/22	-0.1	0.5	20.2	1.7
	01/26/22	02/02/22	0.0	0.5	22.8	1.8
	02/02/22	02/09/22	0.5	0.5	27.7	1.8
	02/09/22	02/16/22	0.7	0.5	27.0	1.8
	02/16/22	02/23/22	0.9 J+ ⁹	0.4 J+ ⁹	21.4	1.8
	02/23/22	03/02/22	0.6	0.5	39.0	2.2
	03/02/22	03/09/22	0.3	0.5	24.0	1.8
	03/09/22	03/16/22	0.1	0.5	17.0	1.6
	03/16/22	03/23/22	0.5	0.5	16.6	1.6
	03/23/22	03/30/22	0.2	0.4	21.0	1.5

Table A-1 continued. Weekly concentrations (in 1×10^{-3} pCi/m³) for gross alpha and gross beta analyses of 47-mm TSP filters for all locations, first quarter, 2022.

Sample Location	Collection Date		Gross Alpha		Gross Beta	
	Start	Stop	Concentration	±2 SD	Concentration	±2 SD
Fort Hall ⁸	12/29/21	01/05/22 ⁷	0.1	0.3	10.2	0.9
	01/05/22	01/12/22 ⁷	0.2	0.3	11.7	1.0
	01/12/22	01/19/22 ⁷	0.8	0.4	32.1	1.5
	01/19/22	01/26/22 ⁷	0.6	0.4	17.7	1.2
	01/26/22	02/02/22 ⁷	0.3	0.4	22.8	1.4
	02/02/22	02/09/22 ⁷	0.4	0.4	23.7	1.4
	02/09/22	02/16/22 ⁷	0.9	0.4	34.1	1.6
	02/16/22	02/23/22 ⁷	1.1 J+ ⁹	0.3 J+ ⁹	19.5	1.2
	02/23/22	03/02/22 ⁷	0.8	0.4	34.3	1.6
	03/02/22	03/09/22 ⁷	0.7	0.4	19.8	1.2
	03/09/22	03/16/22 ⁷	0.3	0.3	14.7	1.2
	03/16/22	03/23/22 ⁷	0.6	0.4	16.1	1.2
	03/23/22	03/30/22 ⁷	0.9	0.5	19.5	1.4
Idaho Falls	12/29/21	01/05/22 ¹	0.2	0.3	23.1	1.3
	01/05/22	01/12/22 ¹	0.5	0.4	24.7	1.3
	01/12/22	01/19/22 ¹	0.8	0.3	46.3	1.5
	01/19/22	01/26/22 ¹	0.5	0.4	34.0	1.5
	01/26/22	02/02/22 ¹	0.5	0.4	38.3	1.7
	02/02/22	02/09/22 ¹	0.6	0.4	34.7	1.6
	02/09/22	02/16/22 ¹	0.9	0.4	48.6	1.8
	02/16/22	02/23/22 ¹	1.0 J+ ⁹	0.3 J+ ⁹	22.0	1.4
	02/23/22	03/02/22 ¹	0.9	0.4	40.5	1.8
	03/02/22	03/09/22	0.7	0.5	35.4	2.1
	03/09/22	03/16/22 ¹	0.3	0.4	17.6	1.4
	03/16/22	03/23/22 ¹	0.4	0.4	15.7	1.2
	03/23/22	03/30/22 ¹	0.7	0.4	23.7	1.4

Note: MDCs typically range from $(0.3 \text{ to } 0.9) \times 10^{-3}$ pCi/m³ for gross alpha and from $(0.9 \text{ to } 1.8) \times 10^{-3}$ pCi/m³ for gross beta.

¹ Faulty rotameter-indicated flow rate. Rotameter stop flow rate was greater than the $\pm 10\%$ error tolerance when compared with a field calibrator. Total volume is an estimate based on elapsed time multiplied by the average of the start and stop flow rate measurements from the calibrator. Results are considered (usable) estimates.

² Power off on arrival. Insufficient sample for valid analysis. Results are rejected (R).

³ NS – No sample. Motor maintenance.

⁴ Motor replaced and restarted on 03/04/2022.

⁵ NS – No sample. Lost filter.

⁶ Damaged filter. Results are (usable) estimates.

⁷ Improper sampling orientation with the filter not fully exposed to the ambient air. Results are considered (usable) estimates.

⁸ Operated by Shoshone-Bannock Tribes.

⁹ The TSP blank gross alpha result for the week of 2/16-2/23/22 minimally exceeded the MDC. The associated TSP gross alpha field results for that week were qualified as biased-high estimates (J+).

Appendix B

Table B.1. Results for all electret ionization chamber (EIC) locations, first quarter, 2022.

Sample Location	Net Corrected Exposure Rate ($\mu\text{R/hr}$) ¹	± 2 SD ($\mu\text{R/hr}$)
Arco	10.4	0.8
Craters of the Moon	11.2	4.9
Big Lost River Rest Area	12.8	1.7
Van Buren Avenue	20.4, 20.8	-
Experimental Field Station	11.7	2.7
Main Gate	15.9	3.4
Atomic City	12.7	1.4
Taber	12.3	3.4
Blackfoot	12.1, 16.8	-
Ft. Hall	8.5, 9.7	-
Idaho Falls	10.3	1.5
Mud Lake/ Terreton	13.6	5.4
Monteview	14.1	3.2
Sand Dunes	12.0	2.1
Howe Met. Tower	13.5	4.6
MP282 -20	11.9	2.1
MP280 -20	10.9	3.3
MP278 -20	10.7	2.7
MP276 -20	12.6	2.4
MP274 -20	11.7	3.8
MP272 -20	14.0, 14.2	-
MP270 -20	10.8, 12.0	-
MP268 -20	9.4	0.5
MP266 -20	12.4	4.1
MP264 -20	10.5	3.9
MP270 -20/26	13.7	1.3
MP268 -20/26	12.2	3.2
MP266 -20/26	9.4	2.8
MP263 -20/26	12.8	2.8
MP261 -20/26	10.1	4.2
MP259 -20/26	10.7	2.0
MP256 -20/26	7.4	3.0
MFC (EBR II)	15.6	1.3
EBR I	13.1	1.3
RWMC	13.1	1.4
CFA	16.8	3.8
CITRC (PBF)	14.4	1.9
INTEC	18.7	5.2
ATR (TRA)	14.1	0.3
NRF	15.6	3.4
TAN/SMC	13.9	3.7
Mud Lake Bank of Commerce	13.5	2.1
MP43-33	10.2	4.7
MP41-33	11.2	4.9
MP39-33	15.2	3.5
MP37-33	11.3	3.7
MP35-33	10.2	2.4
MP33-33	15.0	5.0
MP31-33	8.6	0.9
MP29-33	9.6	1.9

Table B.1 cont. Results for all electret ionization chamber (EIC) locations, first quarter, 2022.

Sample Location	Net Corrected Exposure Rate ($\mu\text{R/hr}$) ¹	± 2 SD ($\mu\text{R/hr}$)
MP27-33	15.1	5.4
MP25-33	13.5	3.6
MP23-33	14.0	4.3
MP21-33	11.6	4.5
MP19-33	12.7	2.9
MP14-33	11.0	3.0
MP11-33	12.9	3.6
MP06-33	9.3	1.2
MP03-33	10.2	3.9
Base of Howe	11.5	0.5
Rover	14.2	1.6
Hamer	14.5	4.5
Sugar City	15.9	3.9
Roberts	13.9	3.7
Big Southern Butte	13.3	2.2
T4 North	18.6	2.6
T4 South	9.0, 9.0	-

¹Results are the average of triplicate exposure rate measurements with the associated sample variability (± 2 SD), or the 2 measured exposure rates remaining after removal of an outlying value. One of the triplicate measurements is rejected if it is outside the average of the triplicate measurements ± 2 SD of the historical population variability. Typically, the two most consistent measurements are reported, based on judgment of the data analyst.

Appendix C

Table C-1. List of volatile organic compounds (VOCs) analyzed for water samples.

Analyte	Minimum detectable concentration (MDC) (expressed in µg/L)
Benzene	0.5
Carbon tetrachloride	0.5
Chlorobenzene	0.5
1,4-Dichlorobenzene	0.5
1,2-Dichlorobenzene	0.5
1,2-Dichloroethane	0.5
1,1-Dichloroethene	0.5
cis-1,2-Dichloroethene	0.5
trans-1,2-Dichloroethene	0.5
1,2-Dichloropropane	0.5
Ethylbenzene	0.5
Methylene Chloride	0.5
Styrene	0.5
Tetrachloroethene (PCE)	0.5
Toluene	0.5
1,2,4-Trichlorobenzene	0.5
1,1,1-Trichloroethane	0.5
1,1,2-Trichloroethane	0.5
Trichloroethylene	0.5
Vinyl chloride	0.5
Xylenes (total)	0.5
Bromodichloromethane	0.5
Dibromochloromethane	0.5
Bromoform	0.5
Chloroform	0.5
Bromobenzene	0.5
Bromochloromethane	0.5
Bromomethane	0.5
n-Butylbenzene	0.5
sec-Butylbenzene	1.0
tert-Butylbenzene	0.5
Chloroethane	0.5
Chloromethane	0.5
2-Chlorotoluene	0.5

Table C-1 continued. List of volatile organic compounds (VOCs) analyzed for water samples.

Analyte	Minimum detectable concentration (MDC) (expressed in µg/L)
4-Chlorotoluene	0.5
1,2-Dibromo-3-chloropropane (DBCP)	0.5
1,2-Dibromoethane (EDB)	0.5
Dibromomethane	0.5
1,3-Dichlorobenzene	0.5
Dichlorodifluoromethane	0.5
1,1-Dichloroethane	0.5
1,3-Dichloropropane	0.5
2,2-Dichloropropane	0.5
1,1-Dichloropropene	0.5
cis-1,3-Dichloropropene	0.5
trans-1,3-Dichloropropene	1.0
Hexachlorobutadiene	0.5
Isopropylbenzene	0.5
p-Isopropyltoluene	0.5
Methyl Ethyl Ketone (MEK)	10
Methyl Tert Butyl Ether (MTBE)	0.5
Naphthalene	0.5
n-Propylbenzene	0.5
1,1,1,2-Tetrachloroethane	0.5
1,1,2,2-Tetrachloroethane	0.5
1,2,3-Trichlorobenzene	0.5
Trichlorofluoromethane	0.5
1,2,3-Trichloropropane	0.5
1,2,4-Trimethylbenzene	1.0
1,3,5-Trimethylbenzene	0.5